



Recent ASDEX Upgrade Research in Support of ITER and DEMO

Hartmut Zohm

for the ASDEX Upgrade /EUROfusion MST1 Team*

MPI für Plasmaphysik, Garching, Germany

**see list at the end of the talk*

- ASDEX Upgrade: machine and programme
- Edge: H-mode access and pedestal physics
- Core: transport and MHD stability
- Exhaust: operation at high P_{sep}/R and $P_{\text{rad,core}}/P_{\text{tot}}$
- Scenario development



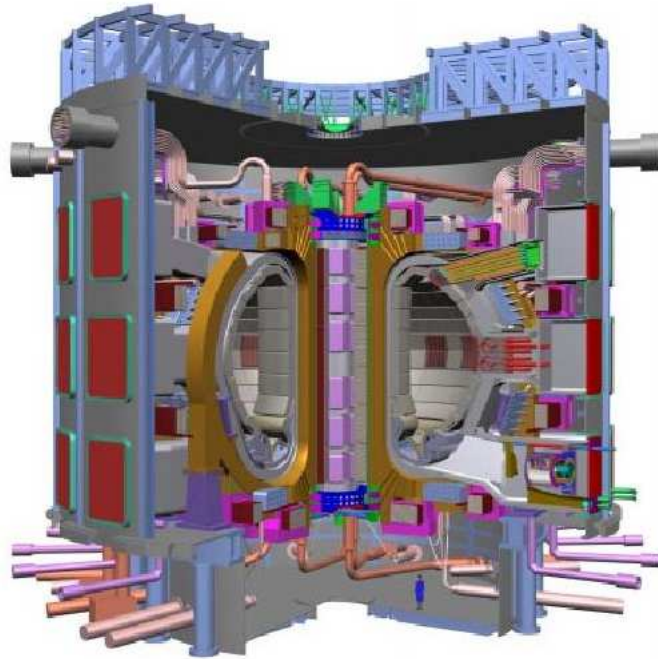
Outline



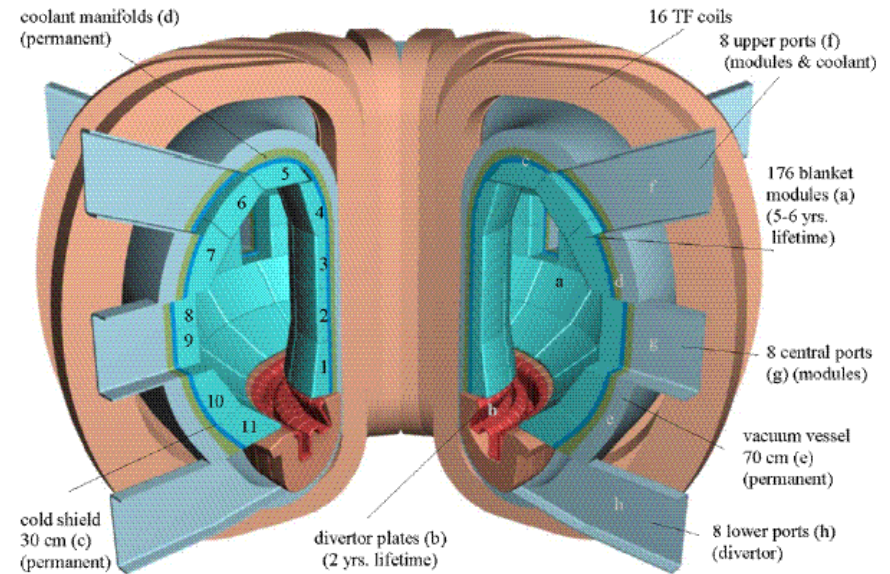
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ITER



DEMO (example)



$Q=10: \beta_N=1.8, H=1, n/n_{GW}=0.85$

$P_{sep}/R = 15 \text{ MW/m}, P_{rad,core}/P_{tot}=0.3$

Large type I ELMs not allowed

Very small number of disruptions

$Q \geq 30: \beta_N=3.5, H=1.2, n/n_{GW}=1.2$

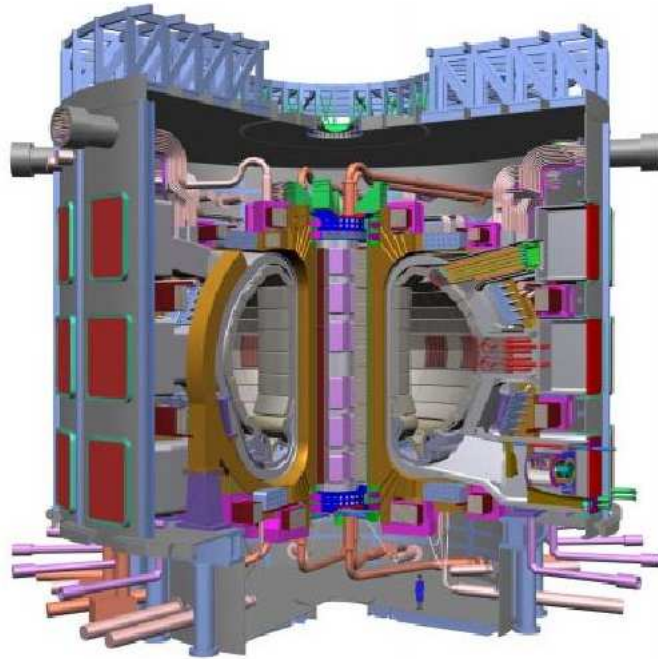
$P_{sep}/R = 15 \text{ MW/m}, P_{rad,core}/P_{tot}=0.75$

No ELMs allowed (?)

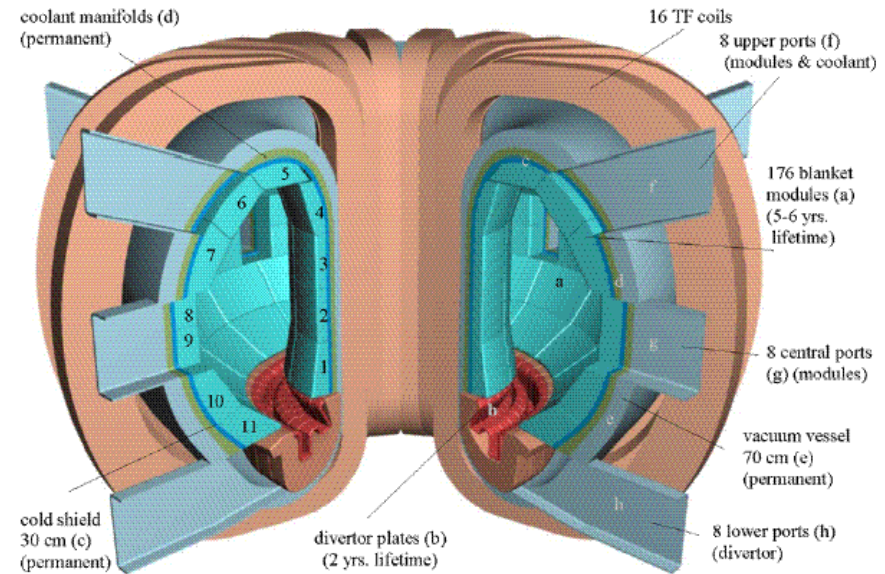
Virtually no disruptions



ITER



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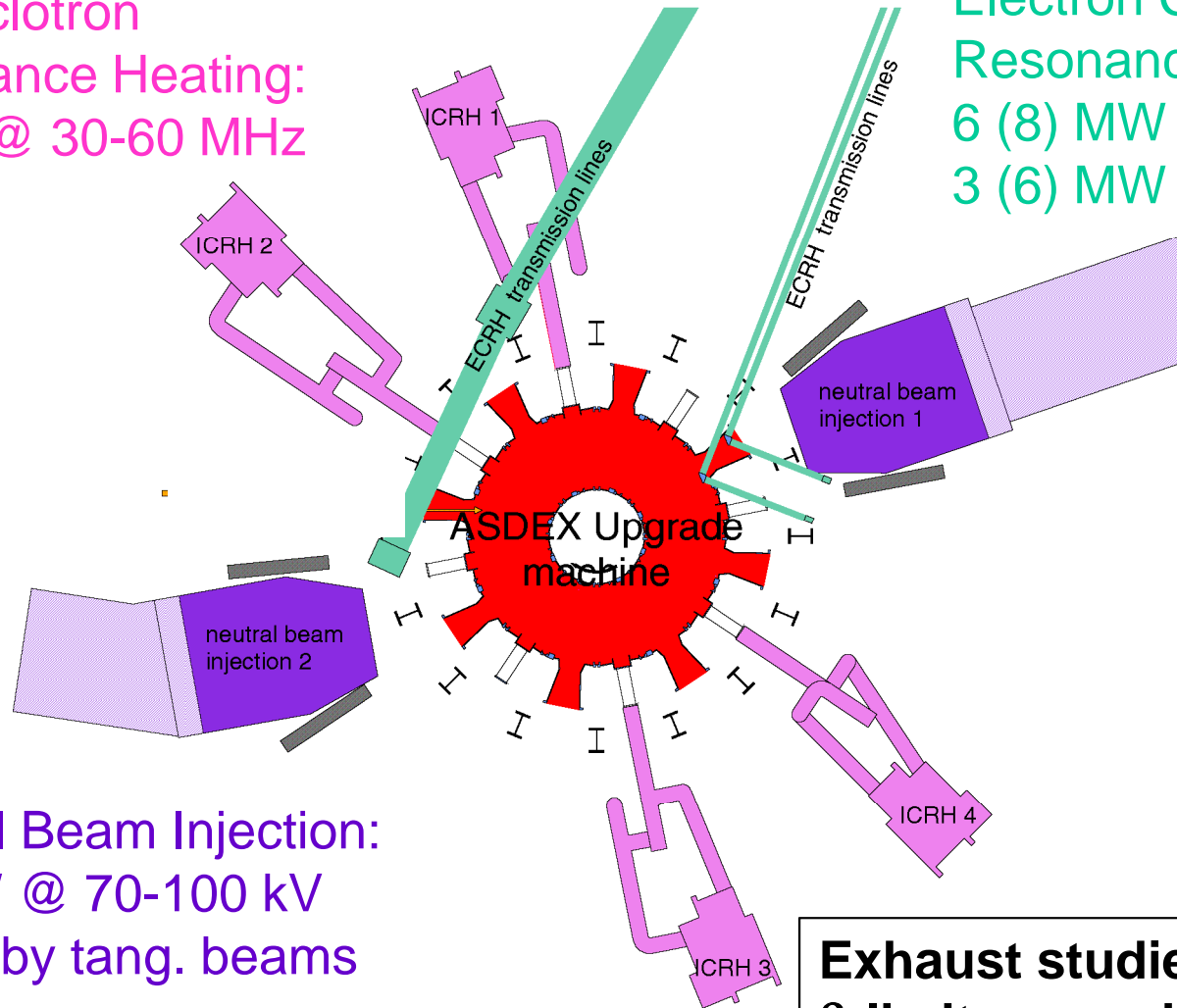


ASDEX Upgrade has a powerful H&CD system



Ion Cyclotron
Resonance Heating:
8 MW @ 30-60 MHz

Electron Cyclotron
Resonance Heating:
6 (8) MW @ 140 GHz
3 (6) MW @ 140/105 GHz

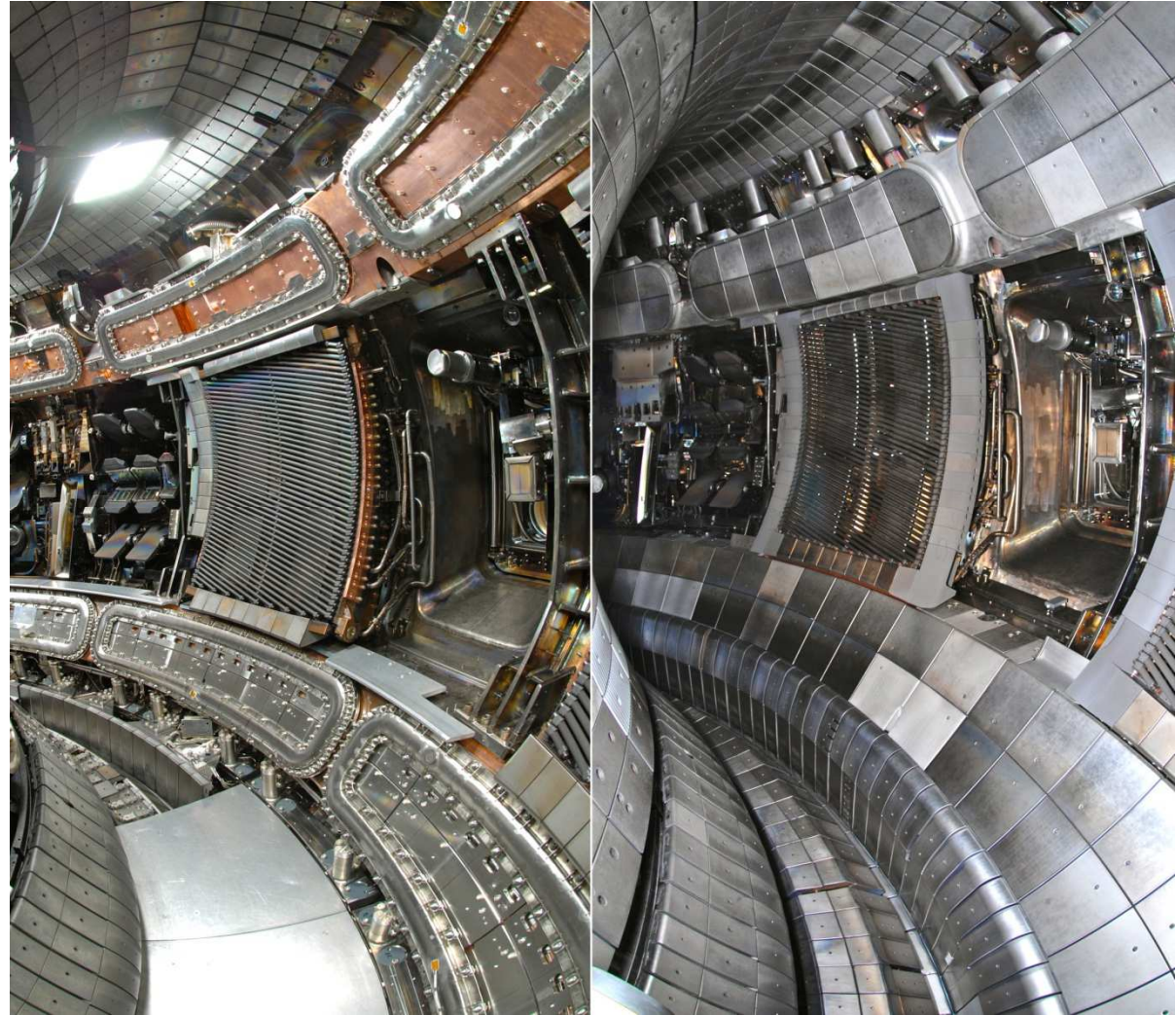
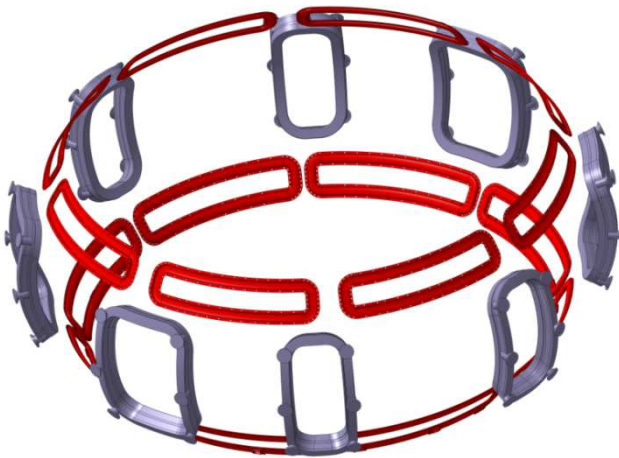
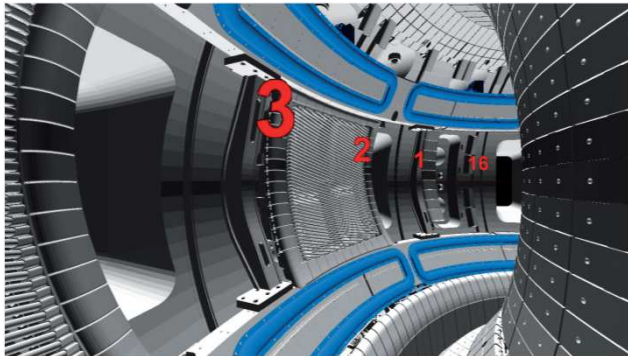


Neutral Beam Injection:
20 MW @ 70-100 kV
NBCD by tang. beams

**Exhaust studies at high P/R
 β -limit accessible at any field
ECCD for MHD control**



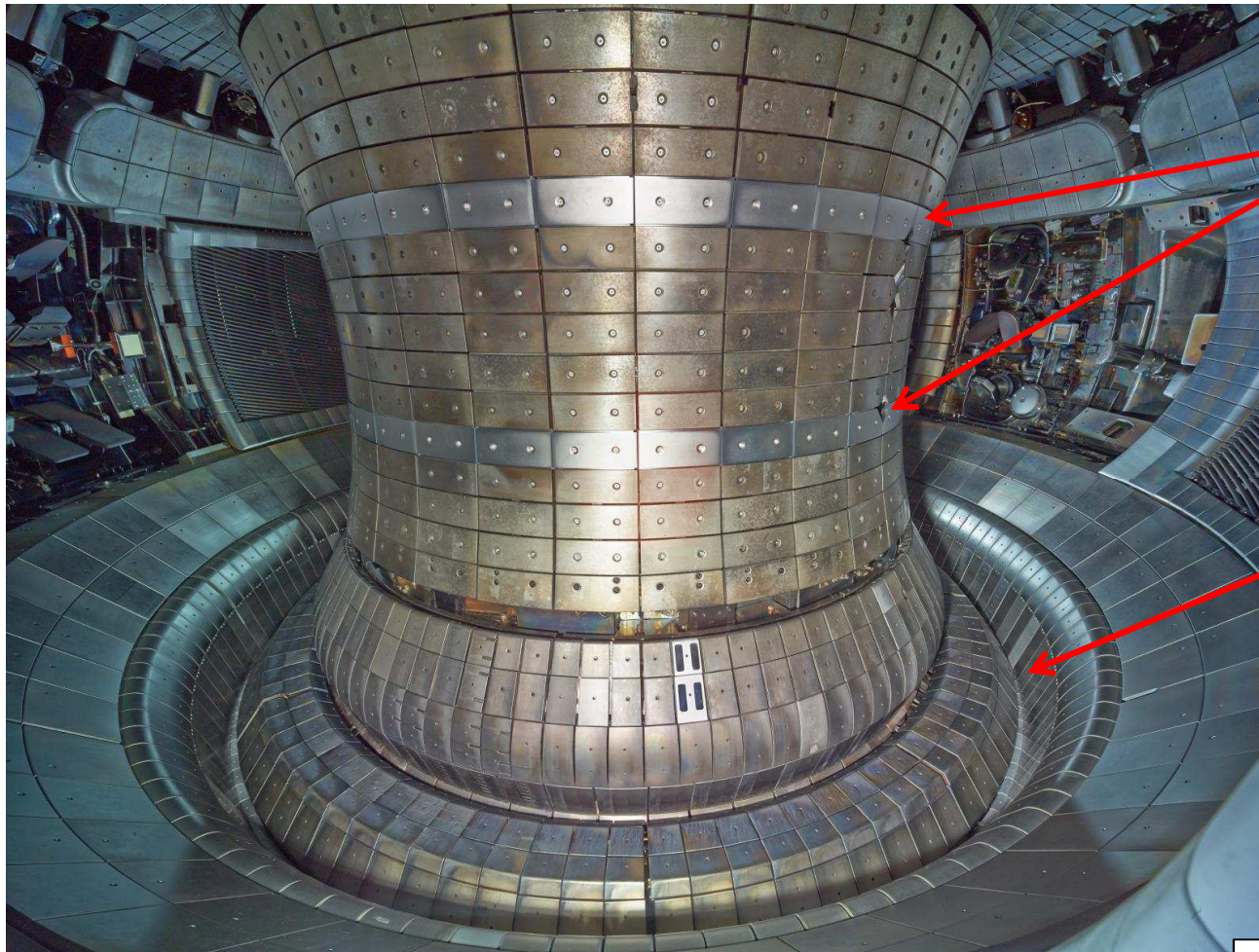
2 x 8 off-midplane saddle coils for MHD control



New: rotating fields up to 150 Hz, continuous poloidal phase scan at constant n



Massive outer W-divertor and Bare Steel Tiles



P92 tiles (chemistry and ferromagnetism similar to EUROFER)

(iron mostly saturated, typical $\mu_r = 1.7$)

massive tungsten tiles

Both enhancements performed reliably without problem during 2014 campaign

A. Herrmann et al., this conference

- + switchable liquid He valve for reduction of pumping (high power scenarios)
- + new divertor manipulator allowing large area sample insertion



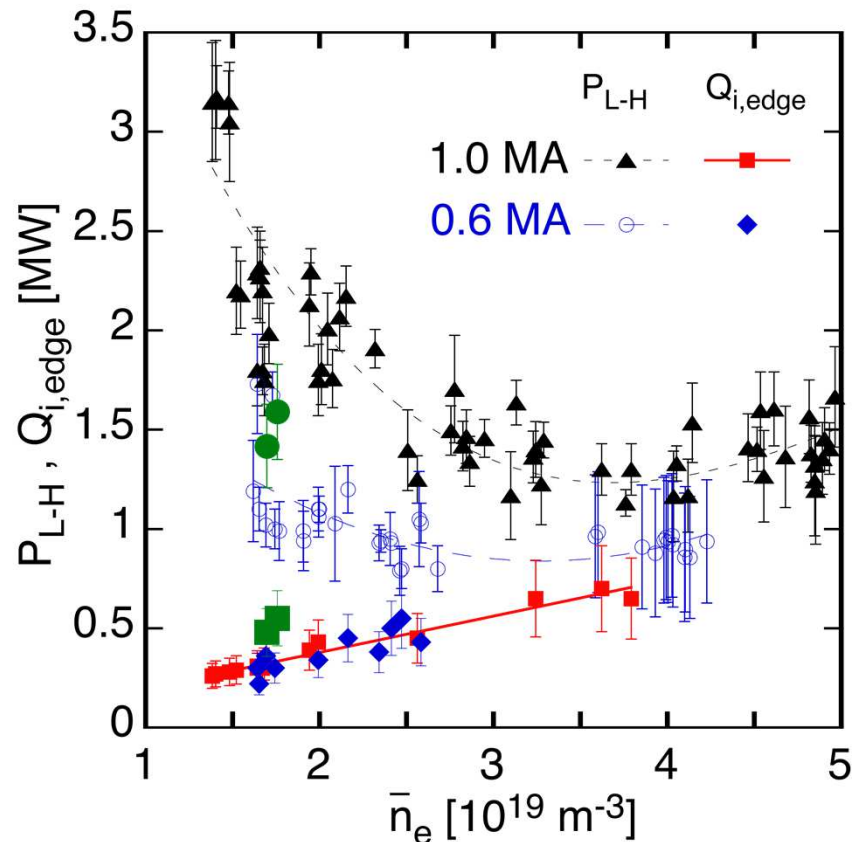
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H-mode operation: low density limit



F. Ryter et al., Nucl. Fusion 2014

Increase of P_{L-H} at low density disappears when plotted versus q_i

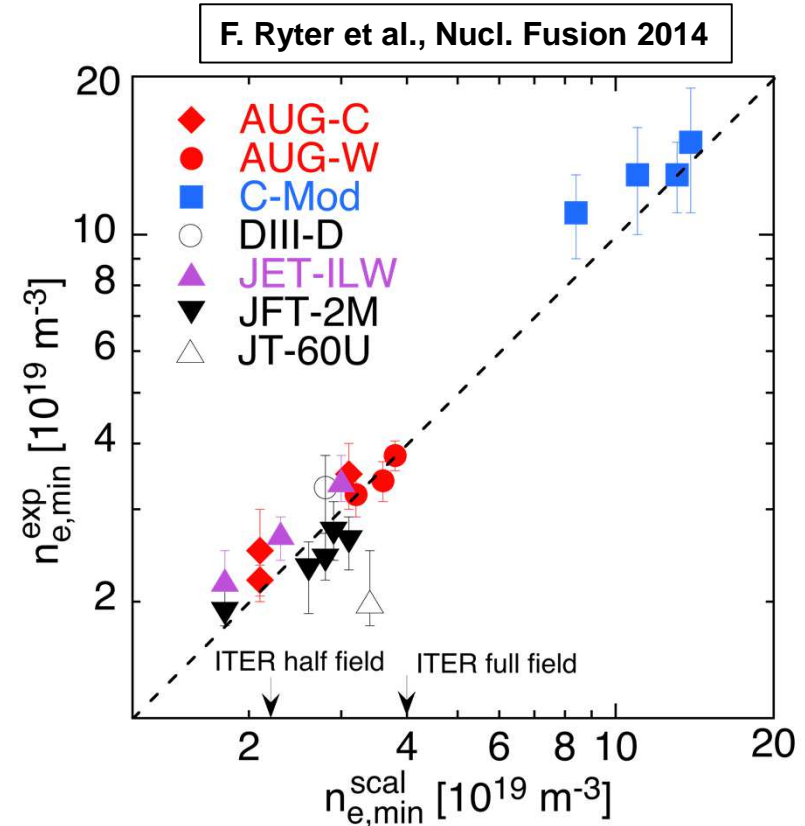
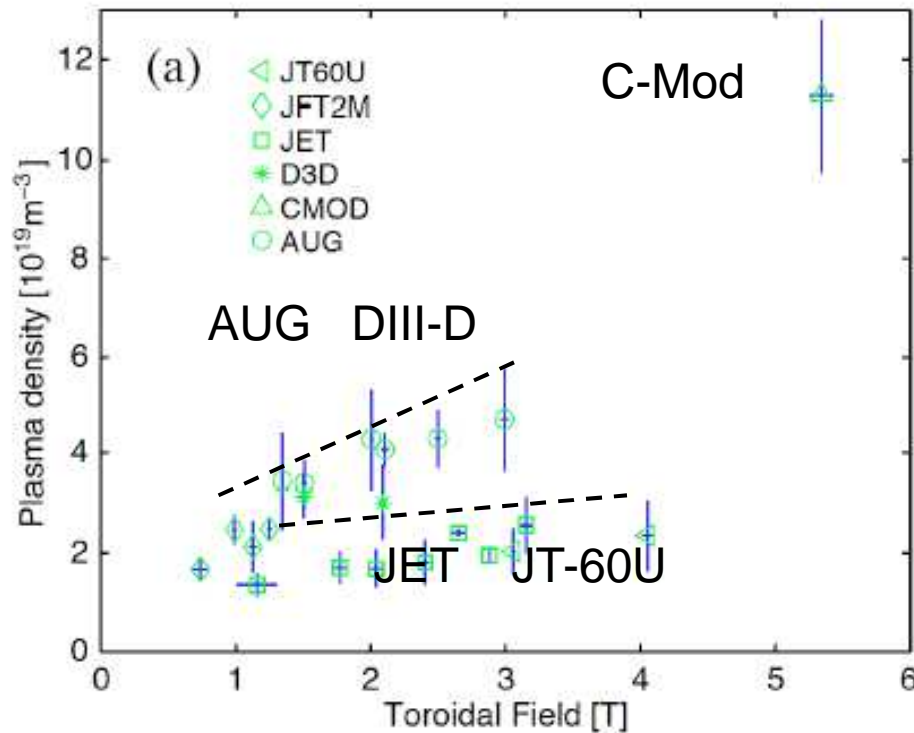
- points towards q_i being main ingredient for edge E_r
- unifies current and heating type dependence at low density

$$E_r \approx \frac{\nabla p_i}{en_i}$$

Reminder: P_{L-H} about 20% lower with all-metal wall, also seen on JET



H-mode operation: low density limit



Transition to low density branch governed by e-i coupling

- assume that $\tau_{ei} / \tau_E \approx \text{const.}$ at $n_{e,\text{min}}$

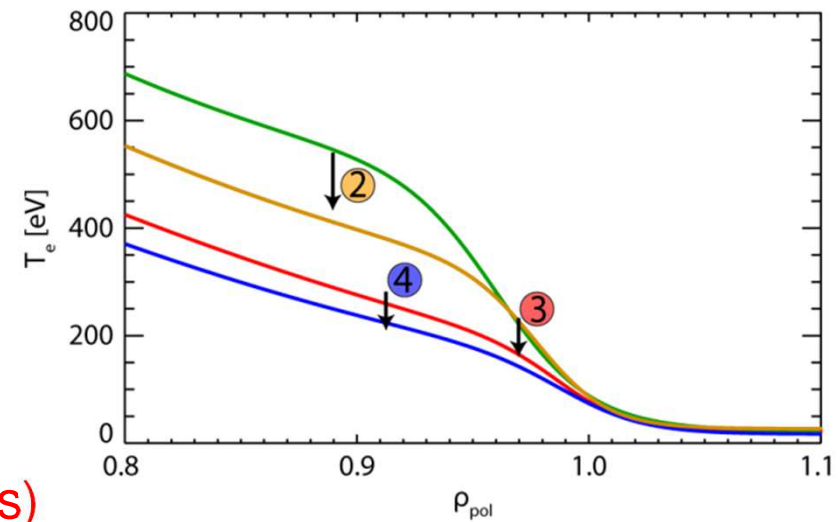
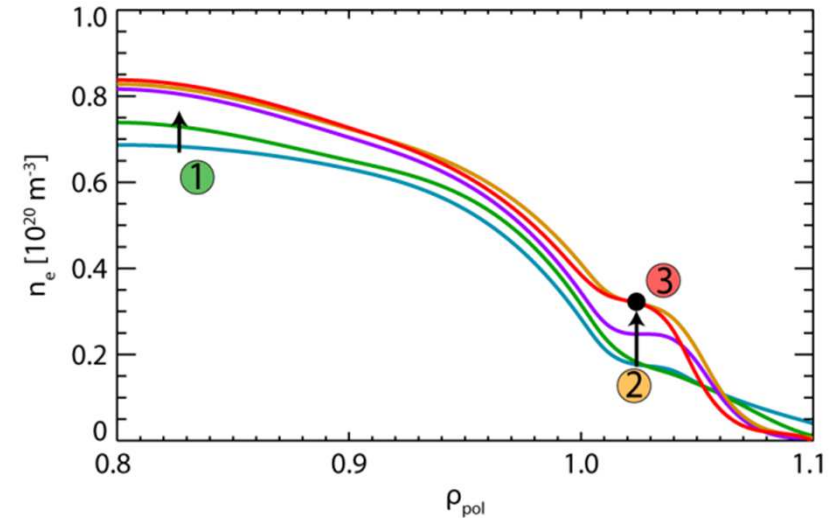
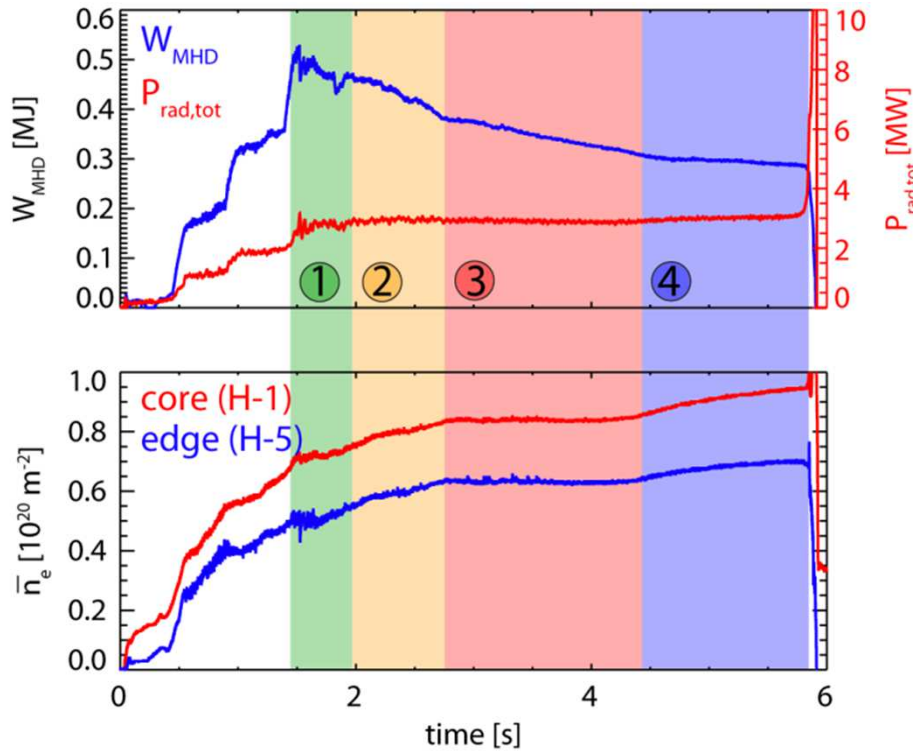
$$n_{e,\text{min}}^{\text{scal}} = 0.7 I_p^{0.34} a^{-0.95} B_T^{0.62} (R/a)^{0.4}$$

- inserting τ_E - and (medium density) P_{LH} -scalings leads to $n_{e,\text{min}}$ scaling

Scaling unifies experimental data, predicts ITER to be in linear regime



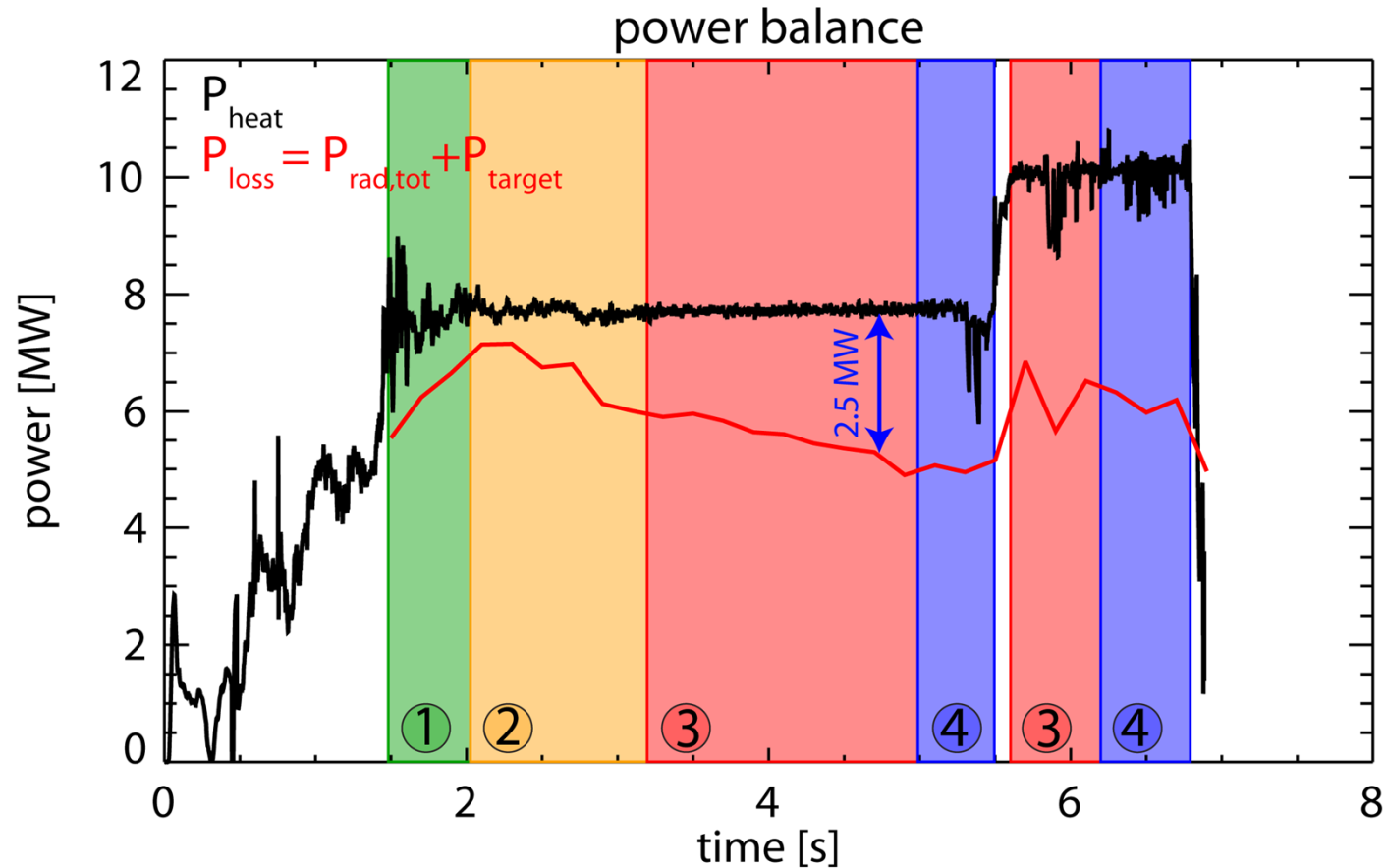
H-mode operation: high density limit



- 1: 'Normal' H-mode (density \uparrow)
- 2: degrading H-mode (only SOL \uparrow)
- 3: H-mode breakdown (pedestal erodes)
- 4: L-mode (density \uparrow , MARFE, disruption)



H-mode density limit by combination of 2 effects



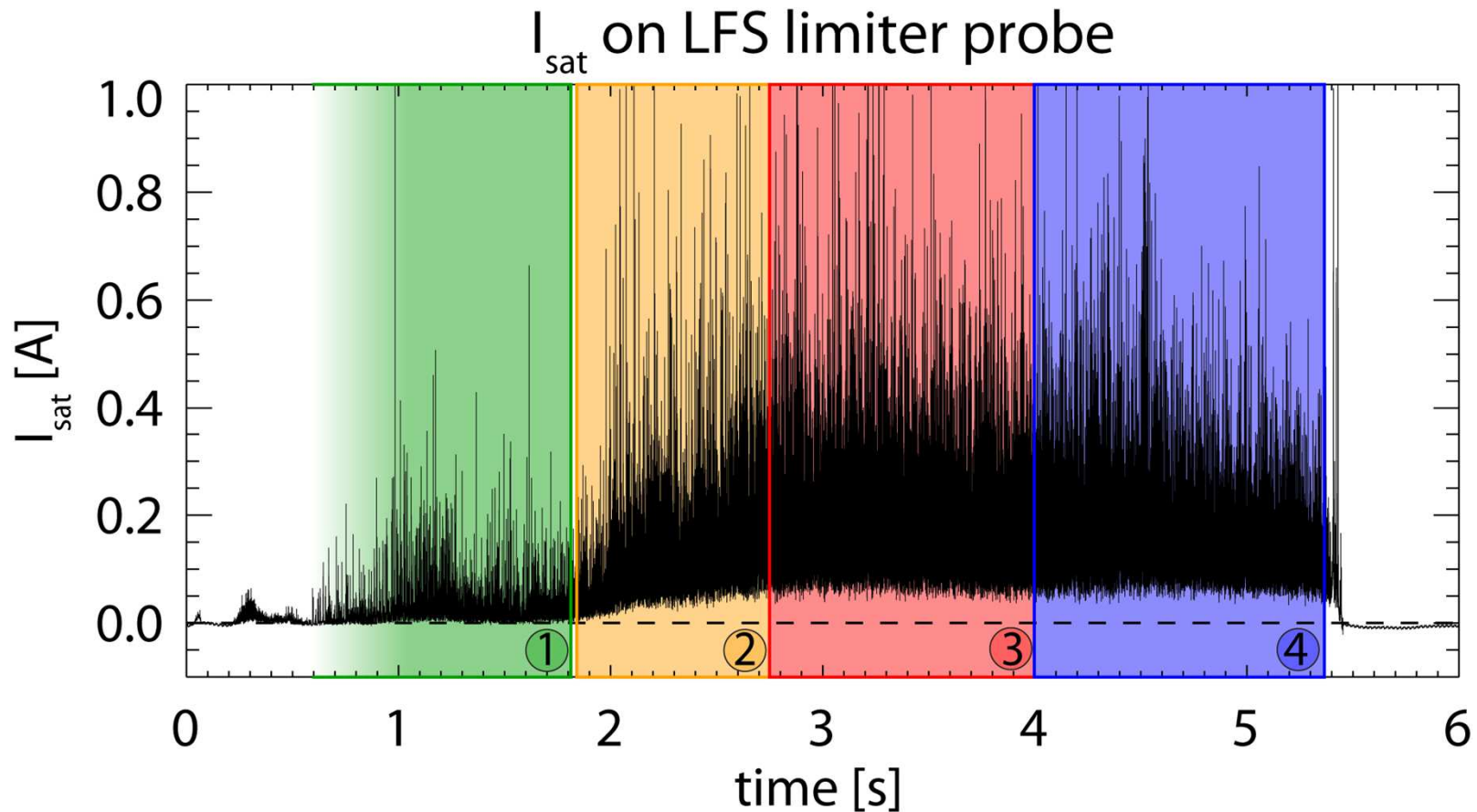
Stagnation of core density build-up due to fuelling limit (source shifts to SOL)

High SOL density leads to strong filamentary transport there

- changed boundary condition at target can increase filament velocity



H-mode density limit by combination of 2 effects



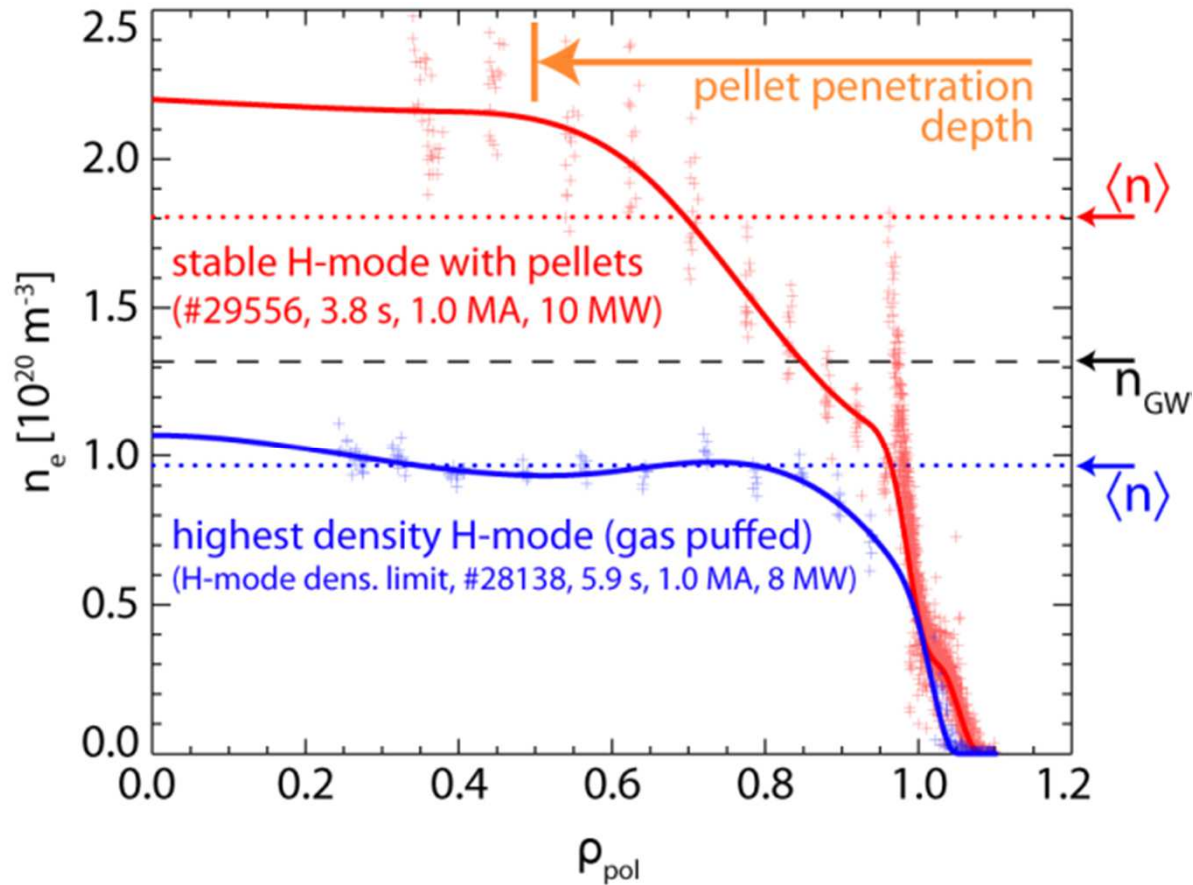
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H-mode operation at $n/n_{GW} > 1$



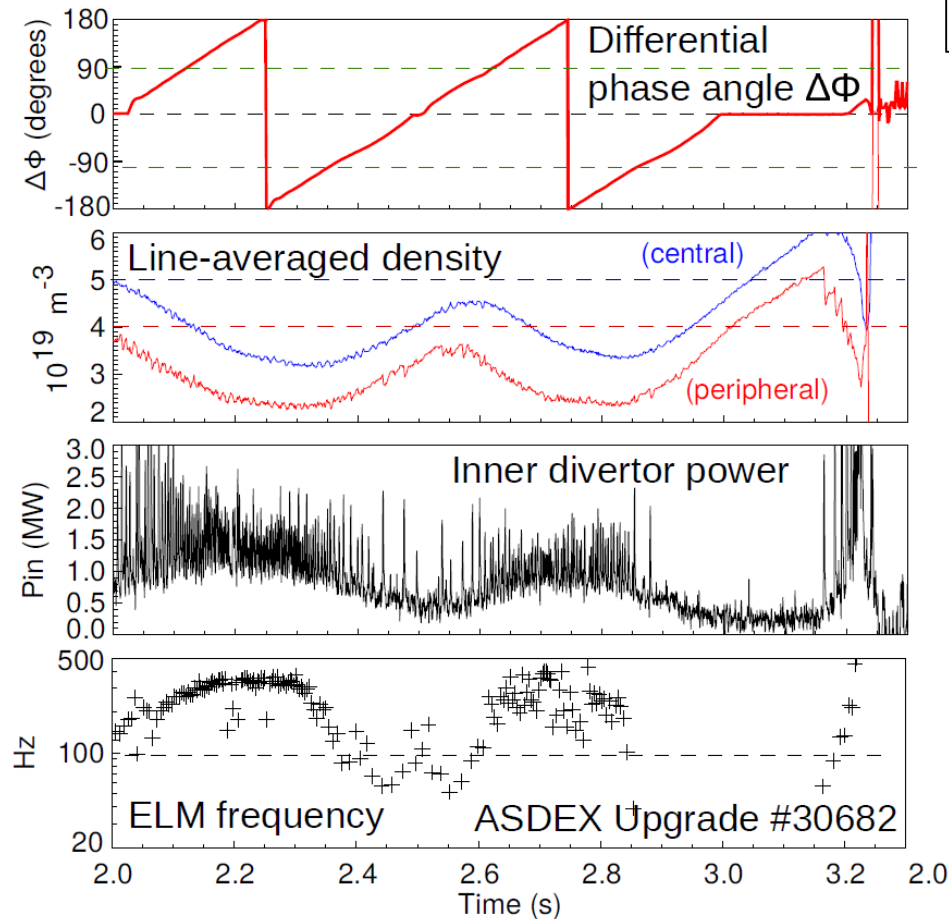
Edge density stays below n_{GW} even with pellets at $n = 1.5 n_{\text{GW}}$

For DEMO: expect strong low collisionality anomalous particle pinch

- DEMO might be able to operate above $n > n_{\text{GW}}$



H-mode operation: ELM Mitigation at low ν^*



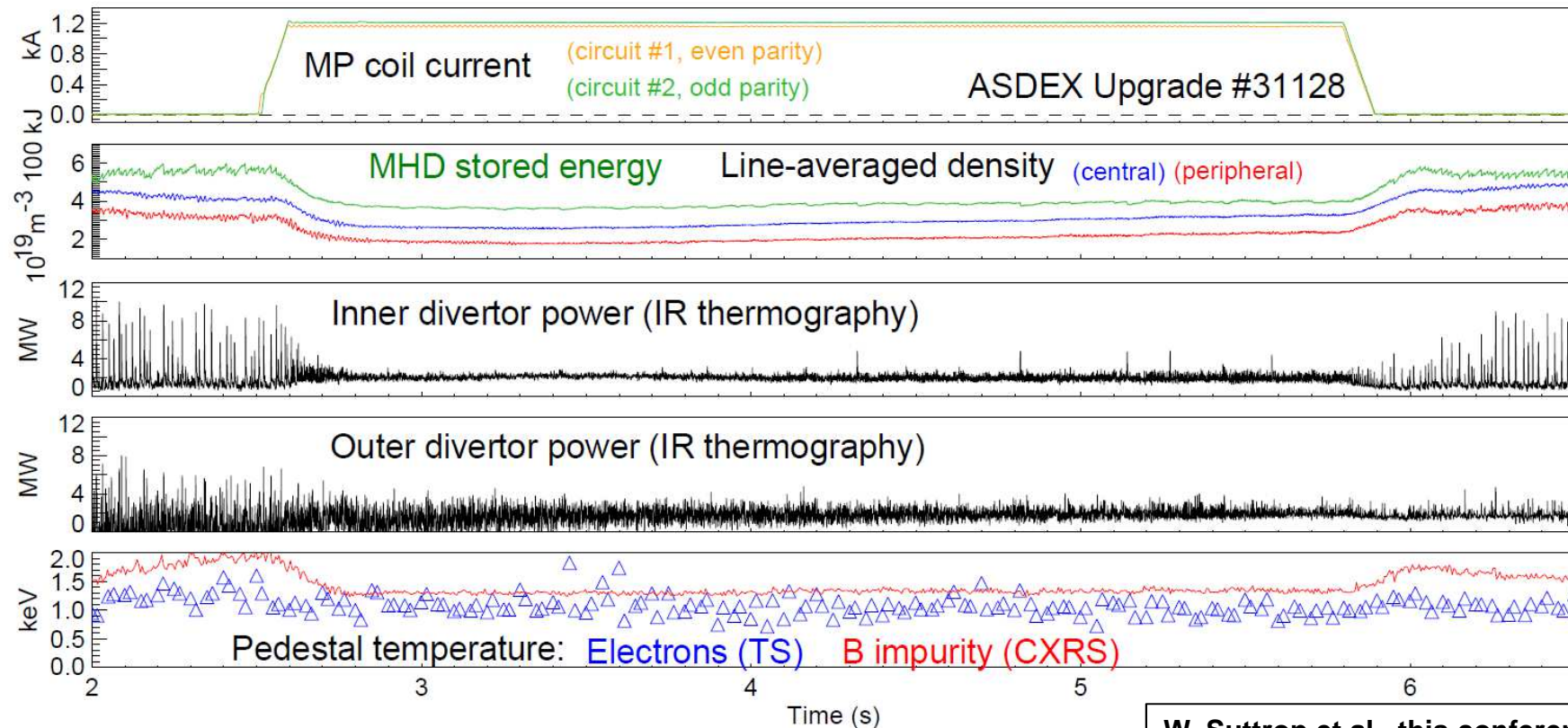
W. Suttrop et al., this conference

Contrary to high ν^* -branch, poloidal spectrum is important

- best ELM mitigation coincides with strongest density pumpout
- note: also 'classical' ELM-free phase can be triggered



H-mode operation: ELM Mitigation at low ν^*



W. Suttrop et al., this conference

At optimum phasing, significant type ELM mitigation is observed

- ELMs still separate events, but much higher frequency, smaller ΔW
- due to strong density pumpout and $T_{i,ped}$ decrease, H is reduced
- optimum mitigation when field is peeling resonant (MARS-F analysis)

A. Kirk et al., this conference



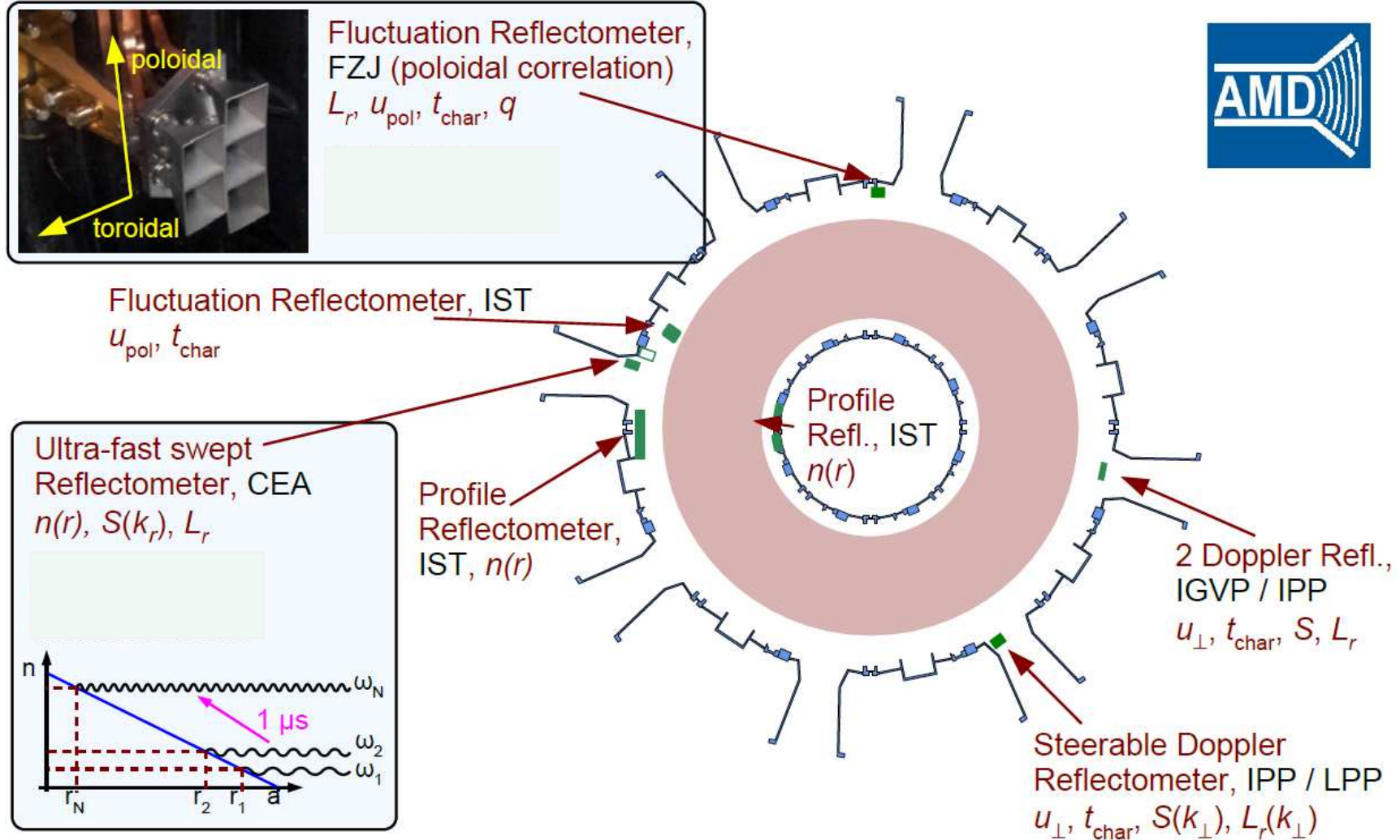
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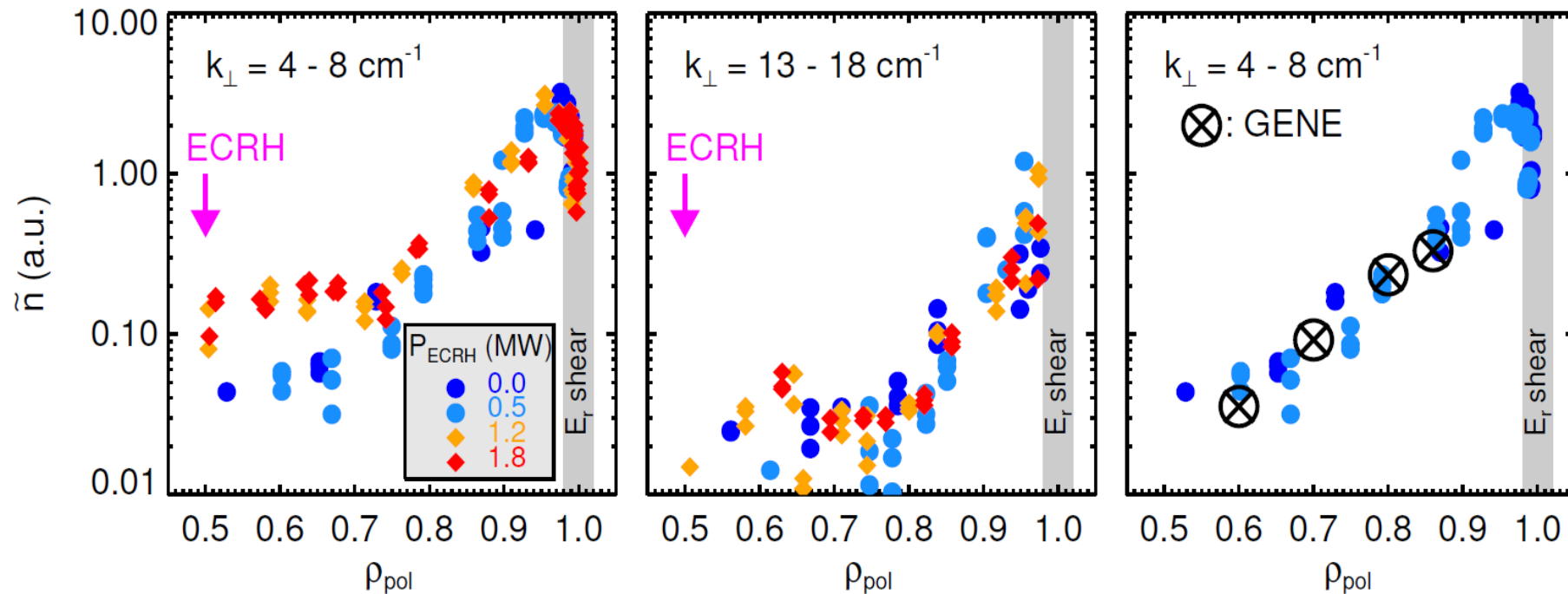
New Microwave Diagnostics for Turbulence Studies



U. Stroth et al., this conference



Core transport: turbulence spectra during ECRH

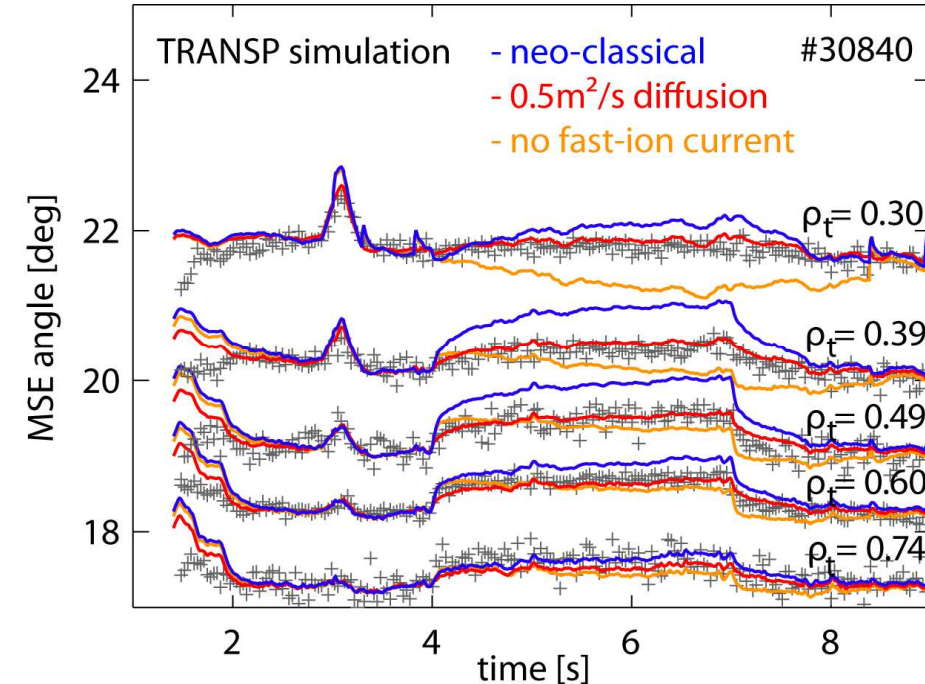
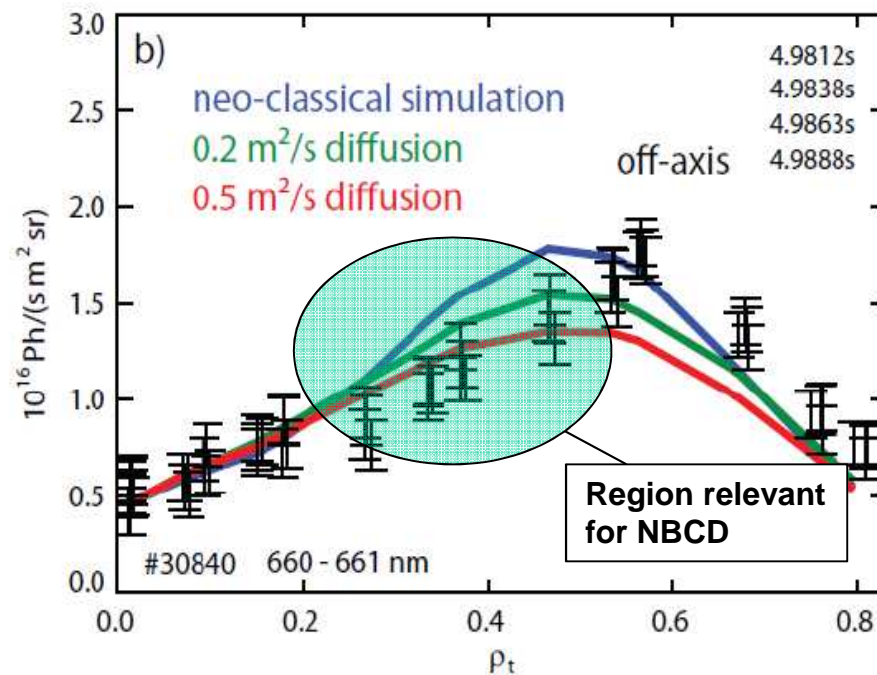


Response of density fluctuations to mid-radius ECRH in H-mode

- low ($k_{\perp} \approx 4-8 \text{ cm}^{-1}$) fluctuations increase while high k_{\perp} does not
- radial amplitude dependence consistent with local flux matched nonlinear GENE simulations that find ITG-regime



Core transport: Fast Ions and NBCD

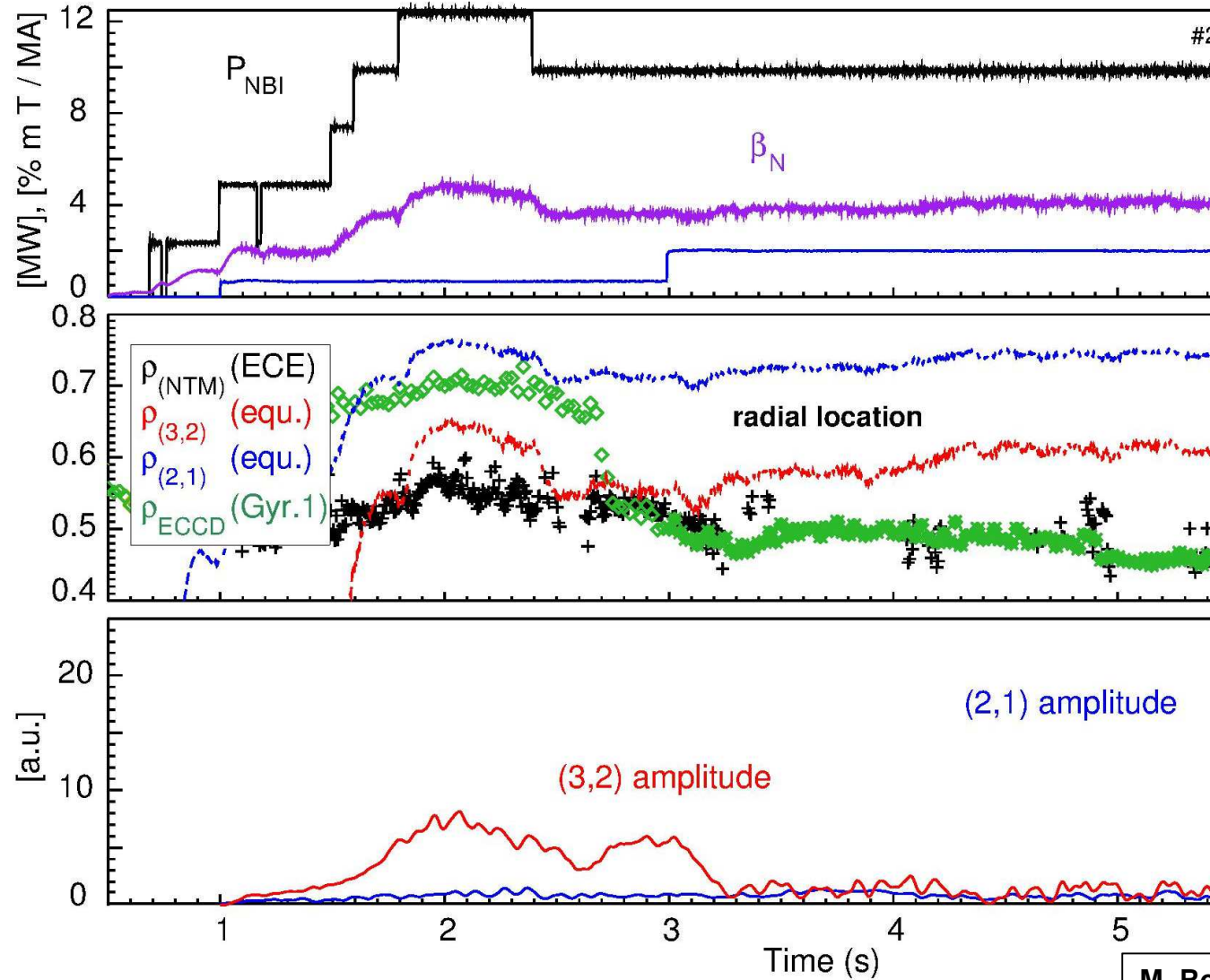


FIDA finds deviation from neo-classical slowing down at high P_{NBI}

- here, also NBCD not consistent with neo-classical prediction
- previous analysis indicated neo-classical slowing down at lower P_{NBI}
- cause not yet clearly identified (some MHD activity present)



Core stability: NTM suppression by ECCD

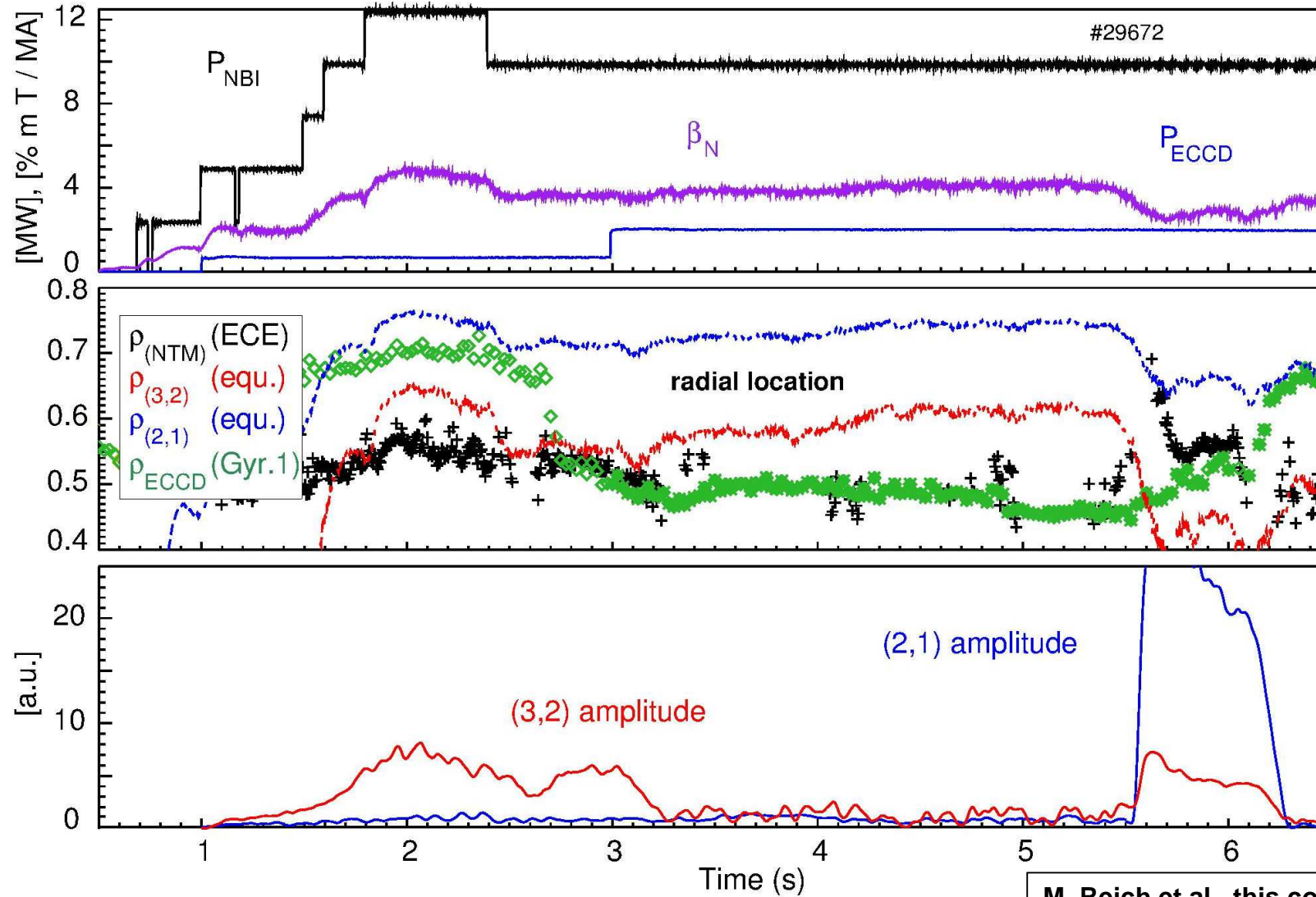


M. Reich et al., this conference

Feedback system targets multiple mode control



Core stability: NTM suppression by ECCD

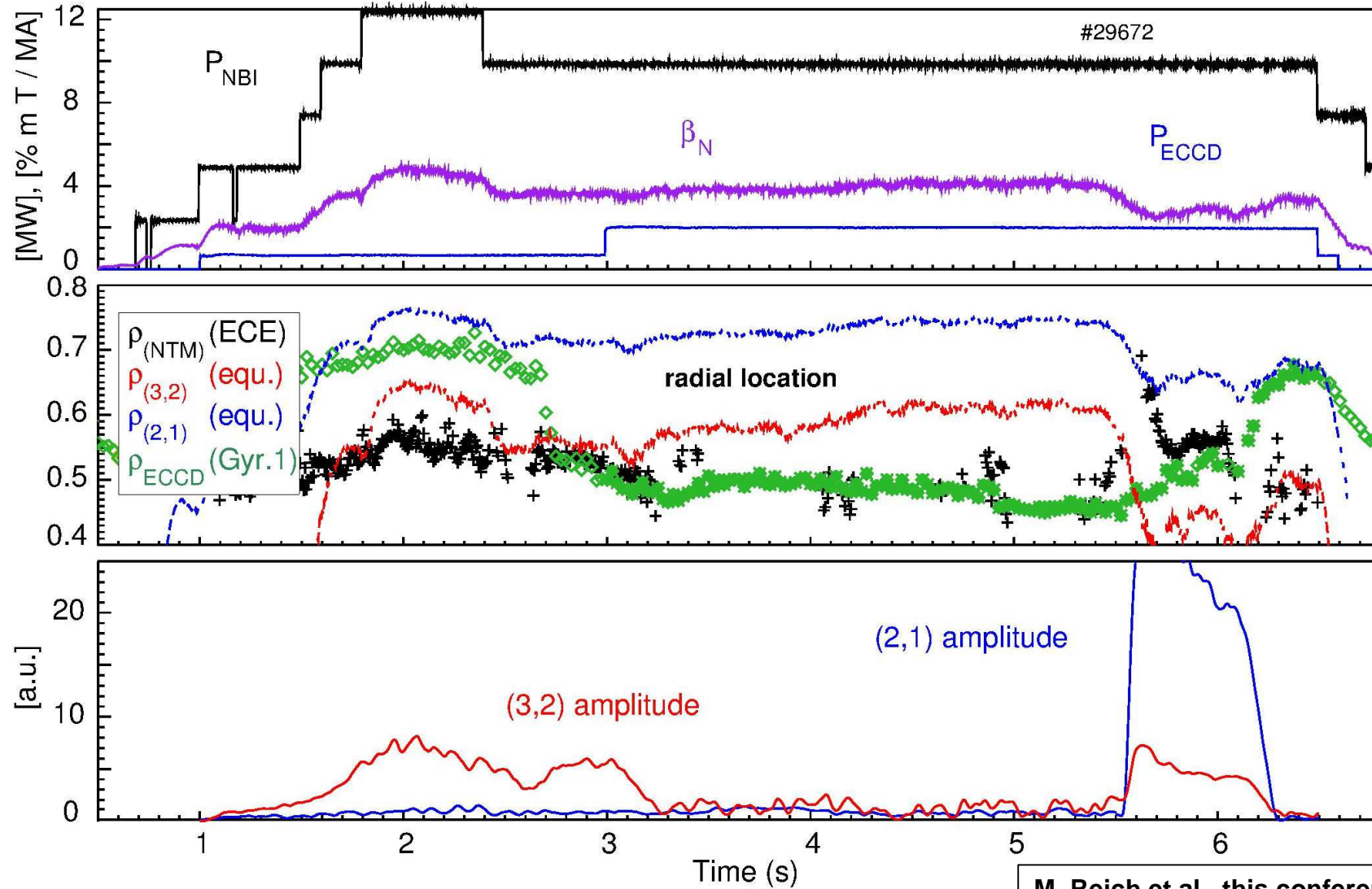


M. Reich et al., this conference

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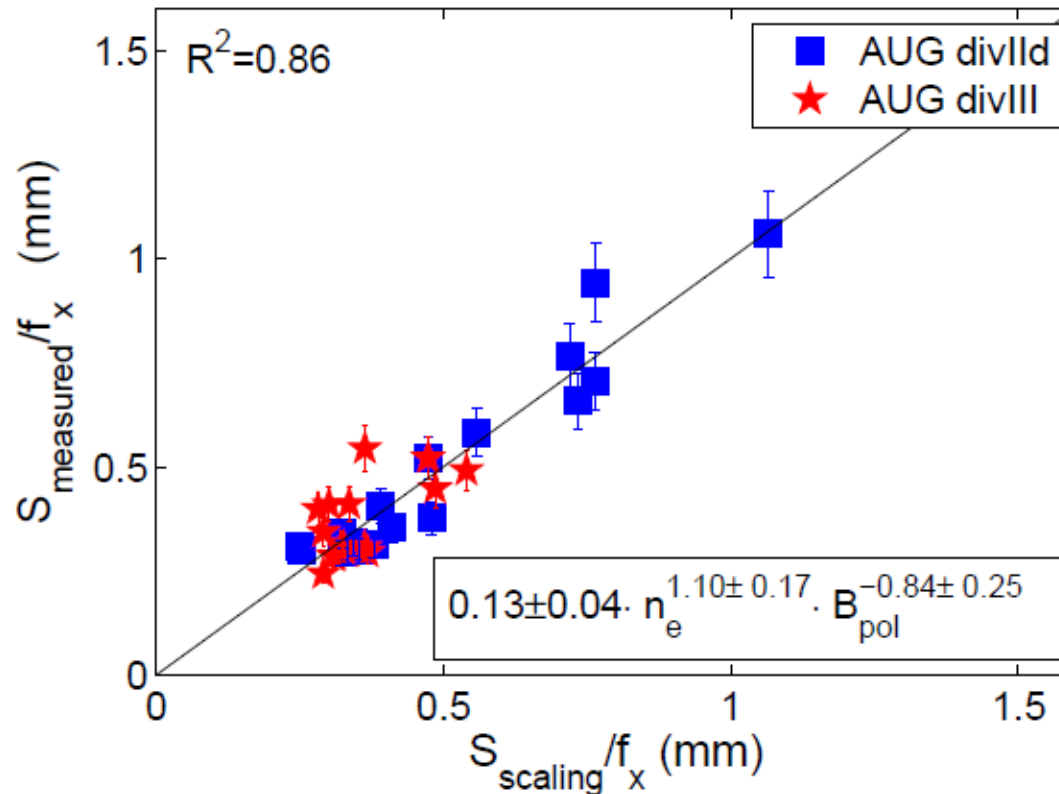
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Broadening of divertor footprint



A. Scarabosio et al., PSI 2014
B. Sieglin et al., PPCF 2013



Midplane λ_q small, scales like ρ_p , not with $R \rightarrow$ figure of merit P_{sep}/R

- broadening by perpendicular transport described by $\lambda_{\text{int}} = \lambda_q + 1.64 S$
- scaling: $S \sim n/B_p$ or $1/T_{\text{target}}$ – consistent with increased $\chi_{\perp}/\chi_{\parallel}$

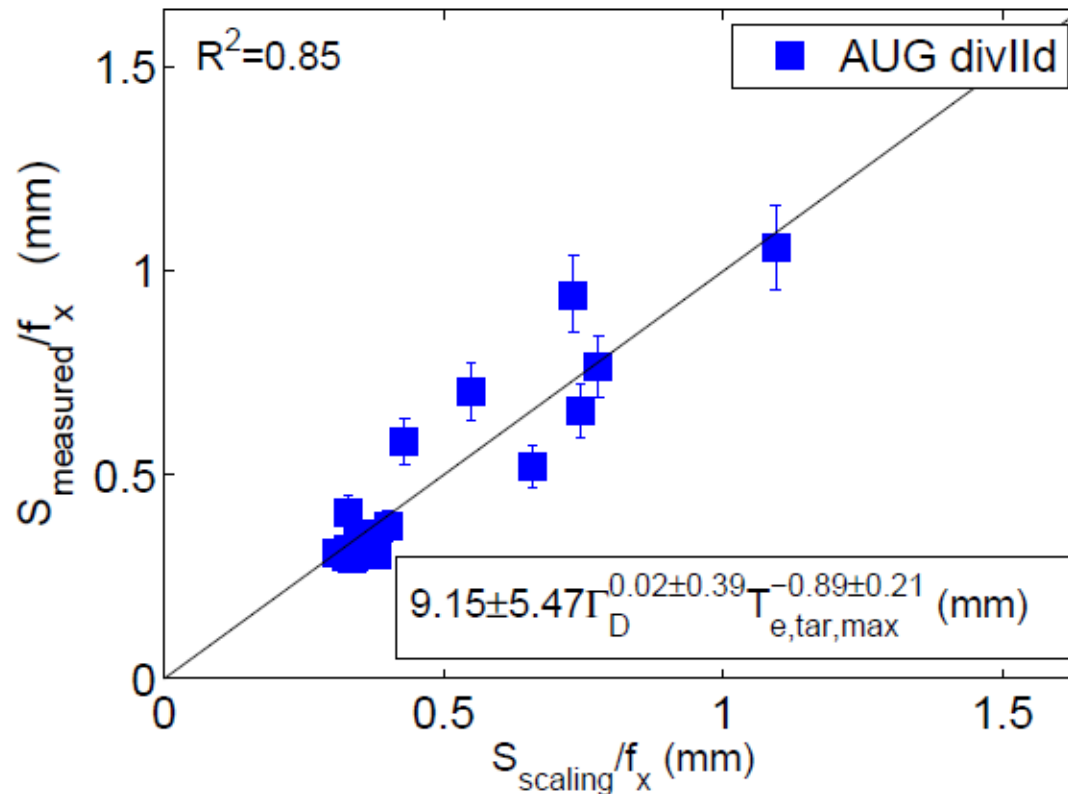
Emphasizes need for detached divertor operation in ITER/DEMO



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A. Scarabosio et al., PSI 2014
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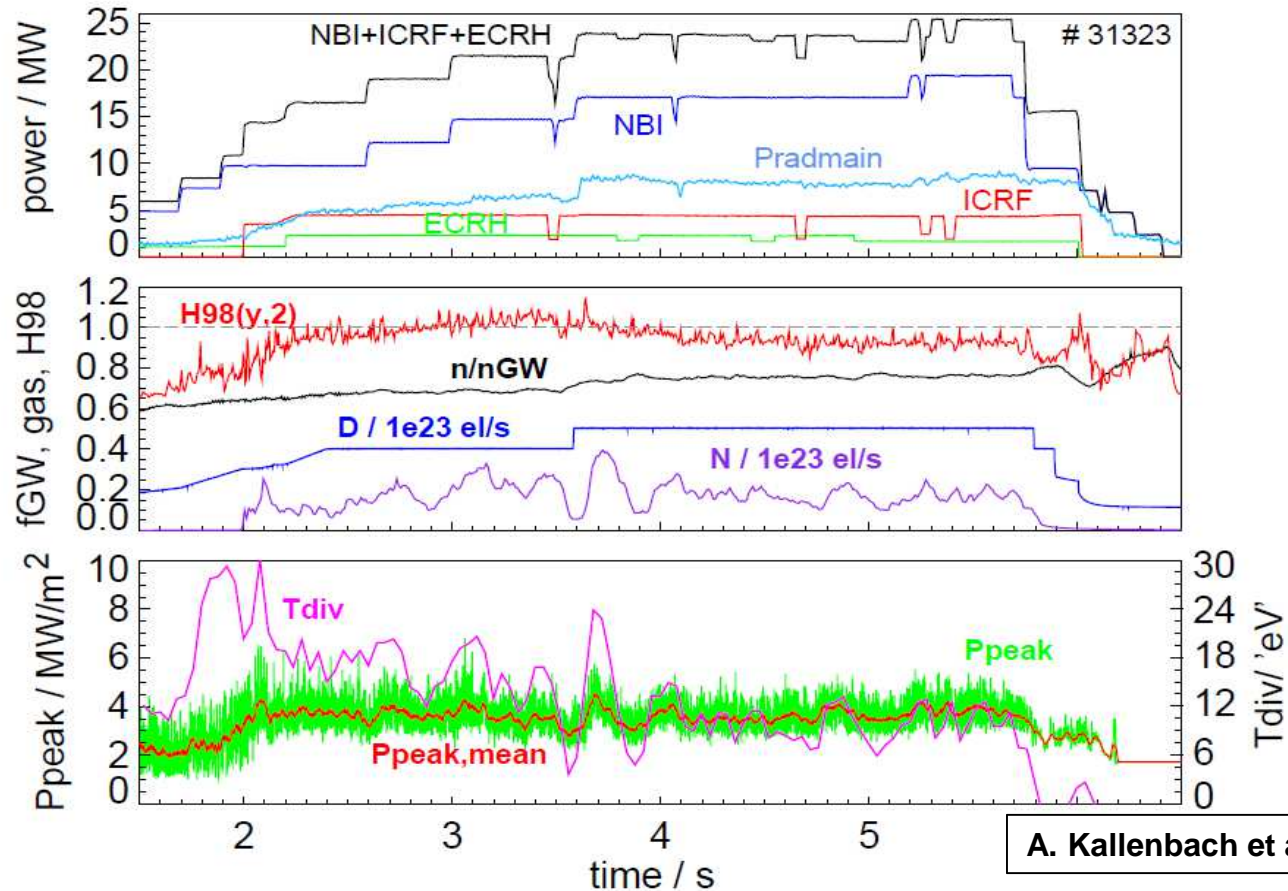
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Emphasizes need for detached divertor operation in ITER/DEMO



Partial detachment at high P_{sep}/R

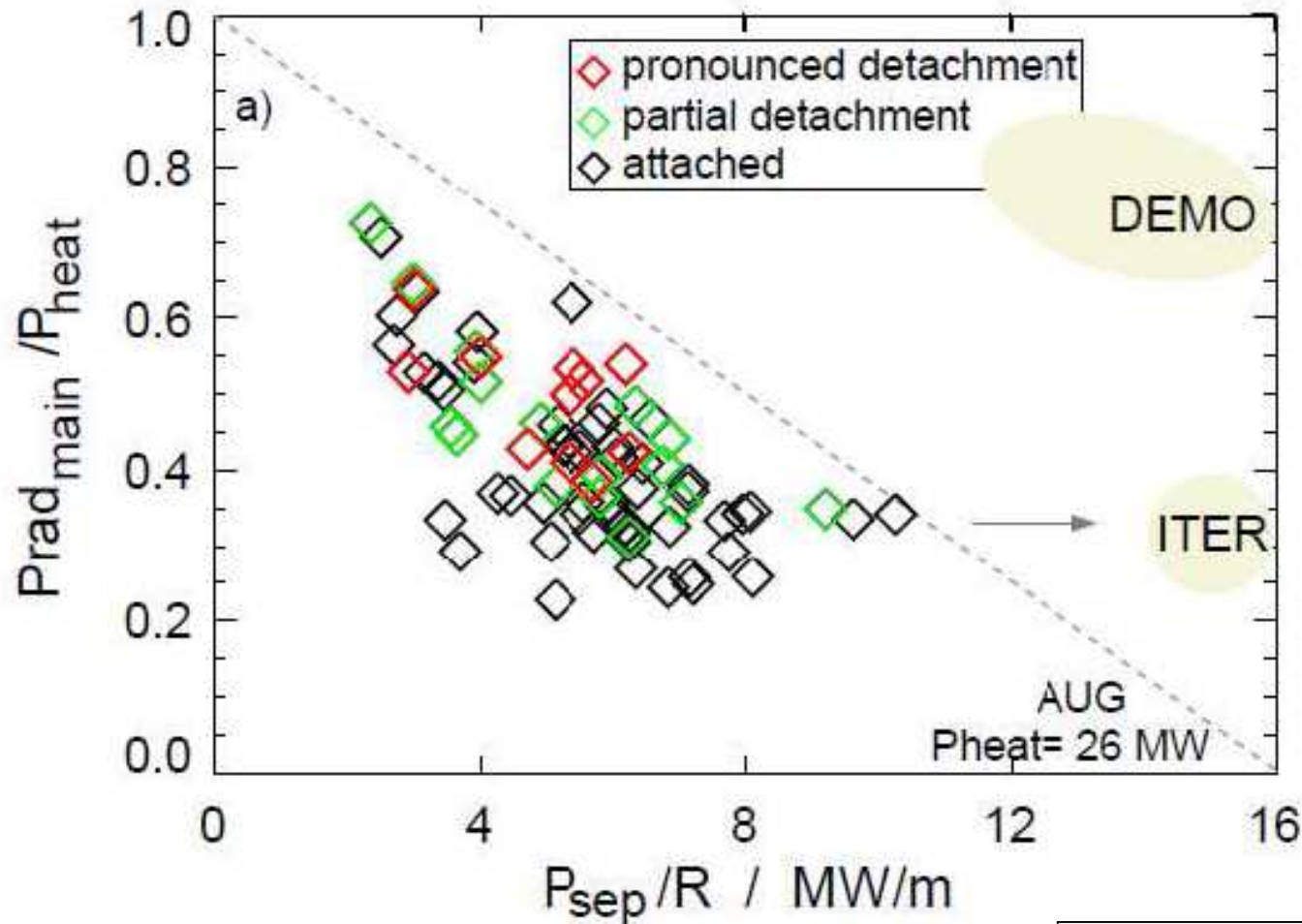


Feedback controlled N-seeding: $q_{div} < 5 \text{ MW/m}^2$ at $P_{heat} = 23 \text{ MW}$

- $P_{sep}/R = 10 \text{ MW/m}$ (2/3 the ITER target) at $H=0.9-1.0$
- with higher stronger seeding, full detachment, but density rises, H drops



Exhaust: present and future capabilities

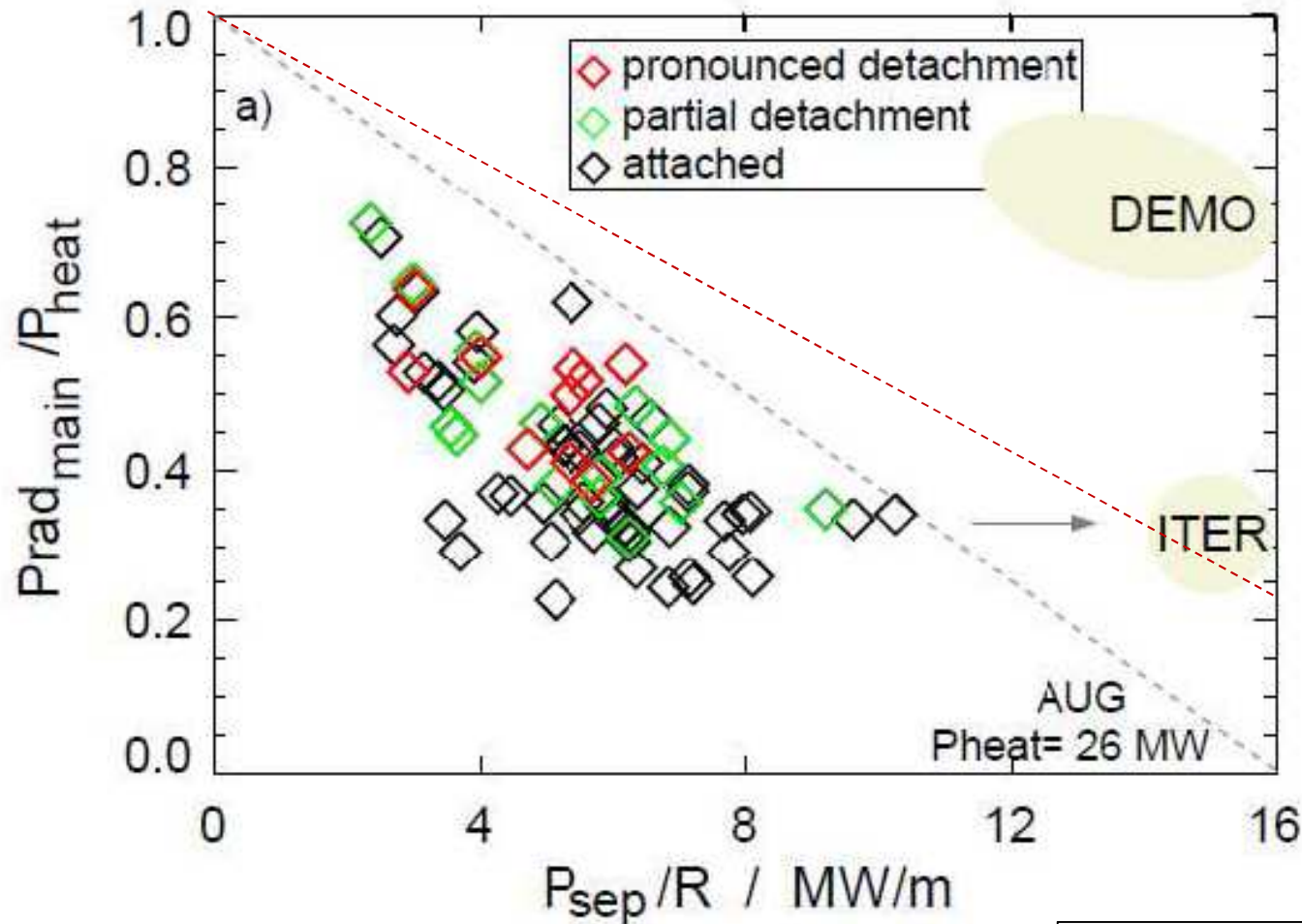


A. Kallenbach et al., this conference

Applying the ITER divertor solution to DEMO, high f_{rad} is needed



Exhaust: present and future capabilities



**AUG with
P_{heat} = 34 MW**

A. Kallenbach et al., this conference

Applying the ITER divertor solution to DEMO, high f_{rad} is needed



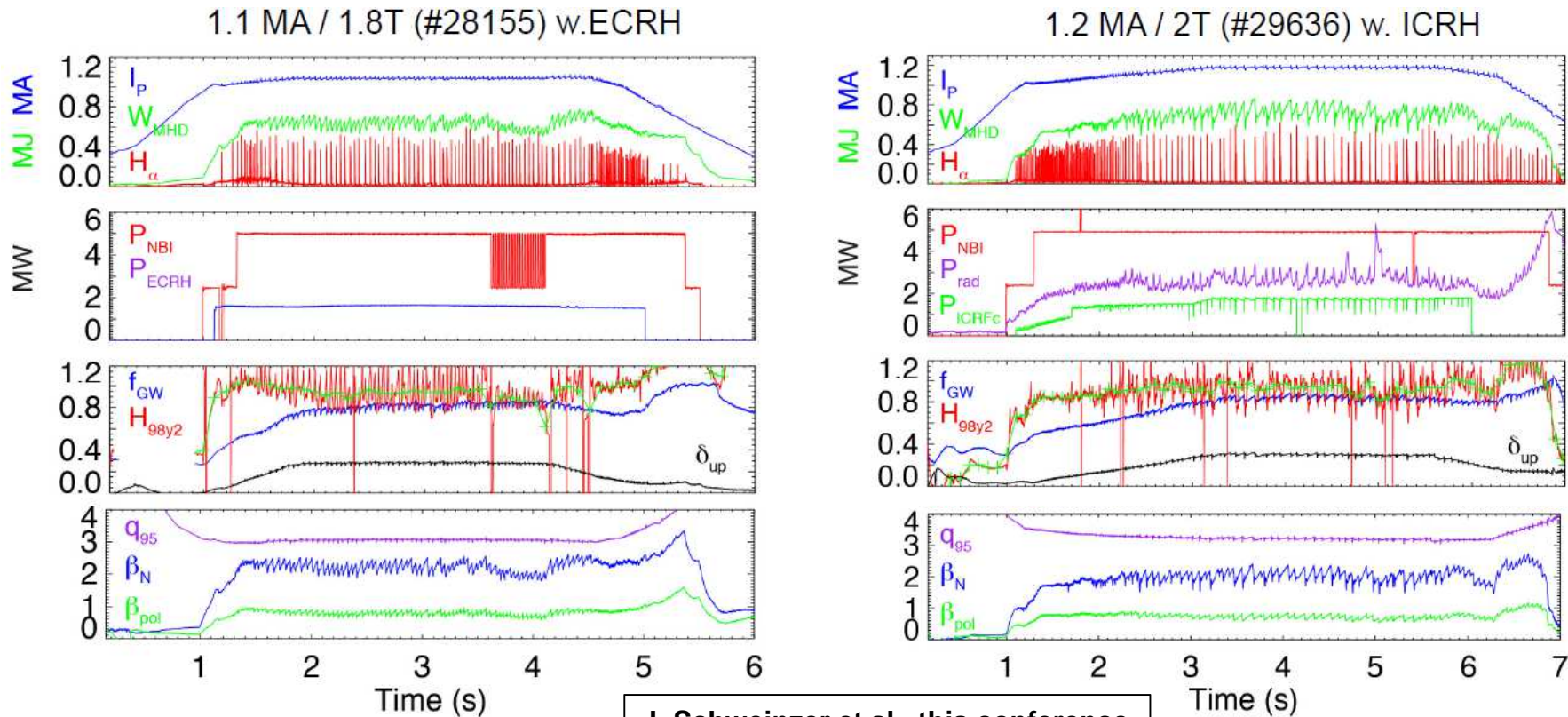
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ITER baseline scenario development



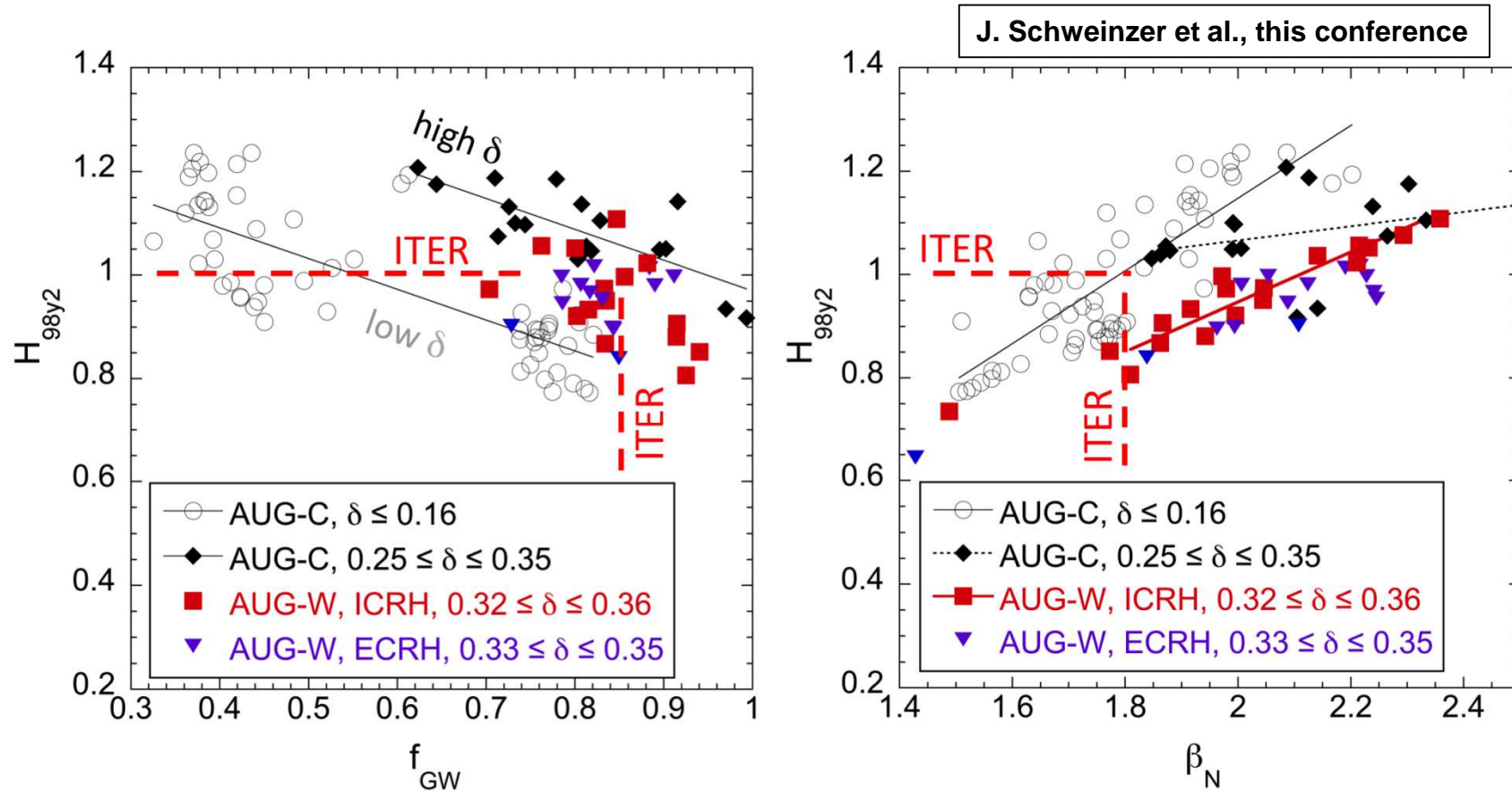
J. Schweinzer et al., this conference

Stable discharges as long as enough gas puff and central heating

- match is in q_{95} , δ , β_N , n/n_{GW} , and hence not in v^* (also not in ρ^*)
- confinement reduced, $H=0.85$ at ITER β_N
- ELMs are large and mitigation techniques do not work reliably



ITER baseline scenario development

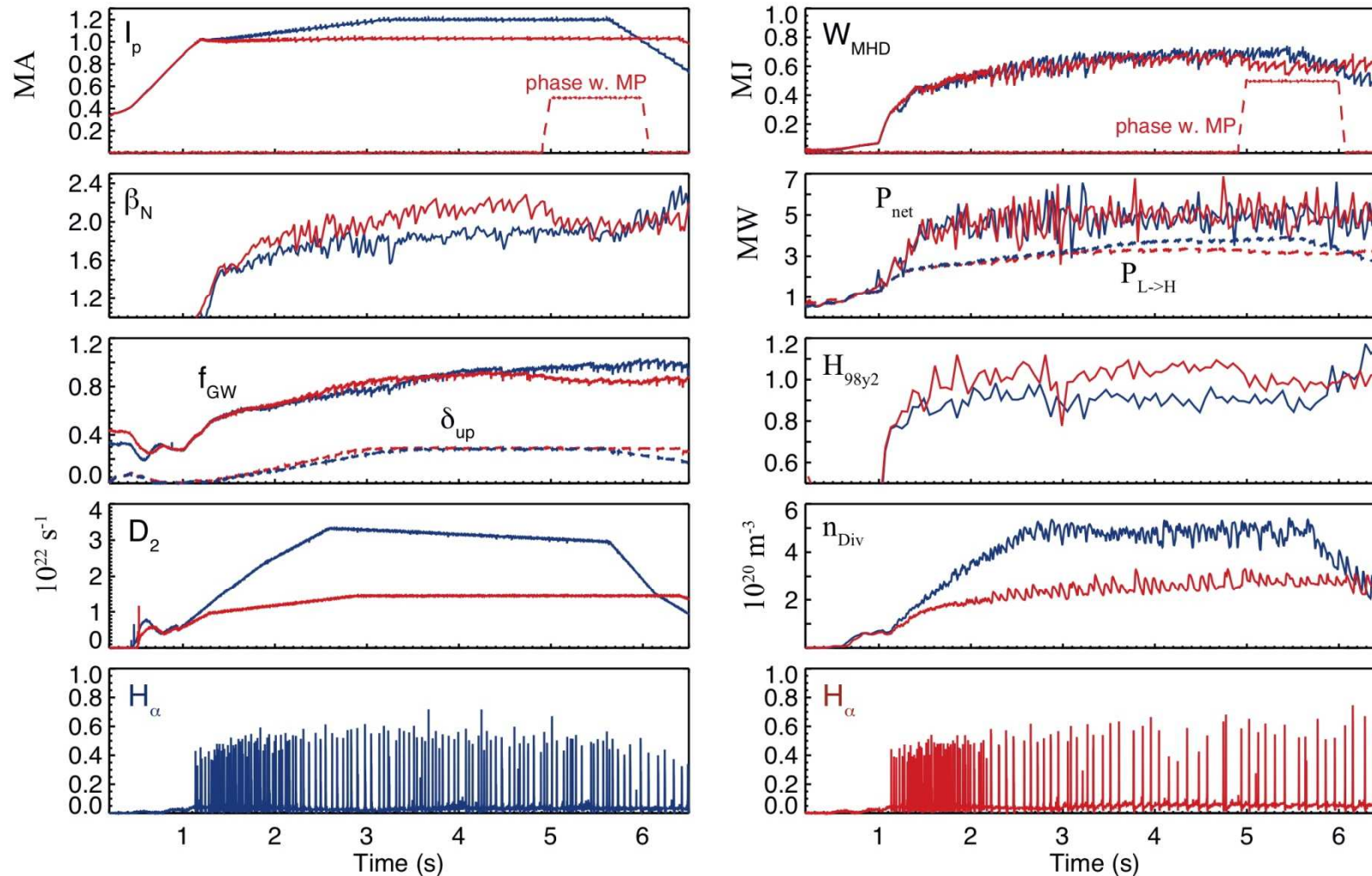


Due to changed operational window, target can only be met at higher β_N

- gas puff needed to keep discharge stable, degrades pedestal
- with higher β_N and N-seeding $H=1$ is recovered (increased edge stability)



ITER baseline scenario development



These findings suggest to move to lower I_p , higher β_N ('improved H-mode')

- first attempt shows same W_{MHD} at 20% lower I_p , target for optimisation



The ASDEX Upgrade / EUROfusion MST1 Team



J. Ahn¹, L. Aho-Mantila², S. Äkäslompolo², C. Angioni, O. Asunta², M. de Baar³, M. Balden, L. Barrera Orte, K. Behler, J. Belapure, A. Bergmann, J. Bernardo⁴, M. Bernert, M. Beurskens⁵, R. Bilato, G. Birkenmeier, V. Bobkov, A. Bock, A. Bogomolov³, T. Bolzonella⁶, J. Boom, B. Böswirth, C. Bottereau¹, A. Bottino, F. Braun, S. Brezinsek⁷, F. Brochard⁸, A. Buhler, A. Burckhart, P. Carvalho⁴, C. Cazzaniga⁶, D. Carralero, L. Casali, M. Cavedon, A. Chankin, I. Chapman⁵, F. Clairet¹, I. Classen³, S. Coda⁹, R. Coelho⁴, K. Coenen⁷, L. Colas¹, G. Conway, S. Costea¹⁰, D.P. Coster, G. Croci¹¹, G. Cseh¹², A. Czarnecka¹³, P. de Marné, P. Denner⁷, R. D'Inca, D. Douai¹, R. Drube, M. Dunne, B. Duval⁹, R. Dux, T. Eich, S. Elgeti, K. Engelhardt, K. Ertl, B. Esposito⁶, E. Fable, U. Fantz, H. Faugel, F. Felici¹⁴, S. Fietz, A. Figueredo⁴, R. Fischer, O. Ford, P. Franzen, L. Frassinetti¹⁵, M. Fröschle, G. Fuchert¹⁶, H. Fünfgelder, J.C. Fuchs, K. Gál-Hobirk, M. Garcia-Muñoz¹⁷, B. Geiger, L. Giannone, E. Giovannozzi⁶, C. Gleason-González⁷, T. Görler, T. Goodman⁹, G. Gorini¹¹, S. da Graca⁴, A. Gräter, G. Granucci¹¹, H. Greuner, J. Griebhammer, M. Groth², A. Gude, S. Günter, L. Guimarães⁴, G. Haas, A.H. Hakola¹⁸, T. Happel, D. Hatch, T. Hauff, B. Heinemann, S. Heinzel, P. Hennequin¹, A. Herrmann, J. Hobirk, M. Hölzl, T. Höschen, J.H. Holm¹⁹, C. Hopf, F. Hoppe, A. Houben, V. Igochine, T. Ilkei¹¹, W. Jacob, A.S. Jacobsen¹⁹, J. Jacquot, M. Janzer, F. Jenko, T. Jensen¹⁹, C. Käsemann, A. Kallenbach, S. Kálvin¹², M. Kantor⁷, A. Kappatou³, O. Kardaun, J. Karhunen², S. Kimmig, A. Kirk⁵, H.-J. Klingshirn, M. Kocan, F. Koch, G. Kocsis¹², A. Köhn¹⁶, M. Köppen, J. Kötterl, R. Koslowski⁷, M. Koubiti¹, M. Kraus, K. Krieger, A. Krivska²⁰, D. Kogut¹, A. Krämer-Flecken⁷, T. Kurki-Suonio², B. Kurzan, K. Lackner, F. Laggner²¹, P.T. Lang, P. Lauber, N. Lazányi¹², A. Lazaros²², A. Lebschy, F. Leuterer, Y. Liang⁷, Ch. Linsmeier, A. Litnovski⁷, A. Lohs, N.C. Luhmann²³, T. Lunt, H. Maier, O. Maj, J. Mailloux⁵, A. Mancini¹¹, A. Manhard, K. Mank, M.-E. Manso⁴, M. Mantsinen²⁴, P. Manz, M. Maraschek, E. Markina, C. Martens, P. Martin²⁵, A. Mayer, M. Mayer, D. Mazon¹, P.J. McCarthy²⁶, R. McDermott, G. Meisl, H. Meister, A. Medvedeva, P. Merkel, R. Merkel, V. Mertens, H. Meyer⁵, O. Meyer¹, D. Milanese⁶, J. Miettunen², A. Mlynek, F. Monaco, D. Moseev, H.W. Müller, S. Müller²⁷, M. Münich, A. Nemes-Czopf⁹, G. Neu, R. Neu, V. Nikolaeva⁴, S.K. Nielsen¹⁹, M. Nocente¹¹, B. Nold¹⁶, J.-M. Noterdaeme, M. Oberkofler, R. Ochoukov, T. Odstrcil, G. Papp, H.K. Park²⁸, A. Pau⁶, G. Pautasso, M.S. Pedersen¹⁹, F. Penzel, P. Piovesan²⁵, C. Piron²⁵, B. Plaum¹⁶, B. Plöckl, V. Plyusnin⁴, Y. Podoba, G. Pokol¹², F. Pompon, E. Poli, K. Polozhiy, S. Potzel, R. Preuss, D. Prisiazhniuk, T. Pütterich, M. Ramish¹⁶, C. Rapson, J. Rasmussen¹⁹, S.K. Rathgeber, G. Raupp, D. Réfy⁹, M. Reich, F. Reimold, M. Reinke⁵, T. Ribeiro, R. Riedl, G. Rocchi¹¹, M. Rodriguez-Ramos¹⁷, V. Rohde, J. Roth, M. Rott, F. Rytter, M. Salewski¹⁹, L. Sanchis-Sanchez¹⁷, G. Santos⁴, J. Santos⁴, P. Sauter, A. Scarabosio, G. Schall, K. Schmid, O. Schmitz²⁹, P.A. Schneider, W. Schneider, M. Schneller, R. Schrittwieser¹⁰, M. Schubert, T. Schwarz-Selinger, J. Schweinzer, B. Scott, T. Sehmer, M. Sertoli, A. Shalpegin³⁰, G. Sias⁶, M. Siccinio, B. Sieglin, A. Sigalov, A. Silva⁴, C. Silva⁴, P. Simon, F. Sommer, C. Sozzi¹¹, M. Spolaore⁶, M. Stejner Petersen¹⁹, J. Stober, F. Stobbe, U. Stroth, E. Strumberger, K. Sugiyama, H.-J. Sun, W. Suttrop, T. Szepesi¹², T. Tala², G. Tardini, C. Tichmann, D. Told, L. Tophøj¹⁹, O. Tudisco⁶, U. von Toussaint, G. Trevisan²⁵, W. Treutterer, M. Valovic⁵, P. Varela⁴, S. Varoutis⁷, D. Vezinet, N. Vianello²⁵, J. Vicente⁴, T. Vierle, E. Viezzer, C. Vorpahl, D. Wagner, X. Wang, T. Wauters²⁰, I. Weidl, M. Weiland, A. Weller, R. Wenninger, B. Wieland, M. Wiesinger²¹, M. Willensdorfer, B. Wirlinger, M. Wischmeier, R. Wolf, E. Wolfrum, D. Wunderlich, E. Würsching, Z. Yang, Q. Yu, I. Zammuto, D. Zarzoso, D. Zsche, M. van Zeeland³¹, T. Zehetbauer, M. Zilker, S. Zoletnik¹², H. Zohm

Max-Planck-Institut für Plasmaphysik, 85748 Garching, Germany;

²Tekes, Aalto University, Helsinki, Finland;

⁴CFN, IST Lisbon, Portugal;

⁶C.R.E., ENEA Frascati, CP 65, 00044 Frascati (Rome), Italy;

⁸Institut Jean Lamour, UMR 7193 CNRS, Vandoeuvre, France;

¹⁰ÖAW, University of Innsbruck, Austria;

¹²KFKI, HAS, Budapest, Hungary;

¹⁴Technische Universiteit Eindhoven, The Netherlands;

¹⁶IGVP, Universität Stuttgart, Germany;

¹⁸TEKES VTT, Espoo, Finland;

²⁰ERM/KMS, Brussels, Belgium;

²²Hellenic Republic, Athens, Greece;

²⁴CIEMAT, Madrid, Spain;

²⁶DCU, University College Cork, Ireland;

²⁸Gwacheon, South Korea;

³⁰University of Nancy, France;

¹CEA, Cadarache, France;

³FOM-Institute DIFFER, TEC, Nieuwegein, The Netherlands;

⁵CCFE Fusion Association, Culham Science Centre, UK;

⁷Forschungszentrum Jülich, Germany;

⁹CRPP, Lausanne, Switzerland;

¹¹ENEA, IFP, CNR, Milano, Italy;

¹³Warsaw University of Technology, 00-661 Warsaw, Poland;

¹⁵VR, Stockholm, Sweden;

¹⁷University of Seville, Spain;

¹⁹DTU, Kgs. Lyngby, Denmark;

²¹ÖAW, IAP, TU Wien, Austria;

²³University of California, Davis, CA 95616, USA;

²⁵Consorzio RFX, ENEA, Padova, Italy;

²⁷University of California, San Diego, CA 92110, USA;

²⁹University of Madison, Wisconsin, USA;